NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | LICENSEE CODE: W I P B H 2 (2) | LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) | LICENSE TYPE: 4 1 1 1 1 (4) | CAT: (5)

CON'T 01 | REPORT SOURCE: L (6) | DOCKET NUMBER: 0 5 0 0 0 3 0 1 (7) | EVENT DATE: 0 9 2 9 7 9 (8) | REPORT DATE: 1 0 1 0 7 9 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 At 0520 hours, 9-29-79, with the unit off line and approximately 20 per
03 cent steam flow, the "B" main steam isolation valve (2-CV-2017) failed
04 to close. Past experience indicates that the reverse type check valve
05 would have closed in high steam flow conditions. Therefore, the valve
06 would have performed its safety function had it been required to do so.
07 This event is reportable per T. S. 15.6.9.2.A.9 and was similar to LER
08 79-001/01T-0.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
09	C D (11)	E (12)	B (13)	V A L V E X (14)	C (15)	D (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 9	—	0 0 7	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	F (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	A 5 8 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The packing was loosened and the Atwood-Morrill reverse swing check
11 valve closed with steam flow at 0533 hours, 9-29-79.
12
13
14

15 | FACILITY STATUS: G (28) | % POWER: 0 2 0 (29) | OTHER STATUS (30): N/A | METHOD OF DISCOVERY: A (31) | DISCOVERY DESCRIPTION (32): Operator observation

16 | ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): N/A | LOCATION OF RELEASE (36): N/A

PERSONNEL EXPOSURES
17 | NUMBER: 0 0 0 (37) | TYPE: Z (38) | DESCRIPTION (39): N/A

PERSONNEL INJURIES
18 | NUMBER: 0 0 0 (40) | DESCRIPTION (41): N/A

LOSS OF OR DAMAGE TO FACILITY (43)
19 | TYPE: Z (42) | DESCRIPTION: N/A

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PUBLICITY
20 | ISSUED: N (44) | DESCRIPTION (45): N/A

7910190429

NRC USE ONLY

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GPO 917-926

ATTACHMENT TO LICENSEE EVENT REPORT NO. 79-007/01T-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

At 0120 hours on September 29, 1979, with Unit 2 off line for secondary side plant maintenance, four condenser steam dump valves were opened (approximately 20% steam flow) during the closing operation of the main steam isolation valves. The "B" main steam valve (2-CV-2017) failed to leave the full open position upon the receipt of a close signal. The "A" main steam isolation (2-CV-2018) valve closed fully.

The "B" valve partially closed with some mechanical assistance. The valve was then reopened and with both mechanical assistance and steam flow, it swung to a position one inch short of the fully closed position. The valve was again reopened and the gland stuffing box packing was loosened. Steam flow was initiated, the close signal given and the valve fully closed at 0533 hours, September 29, 1979.

After a recent similar event (see Licensee Event Report No. 79-001/01T-0), the plant staff concluded that the main steam isolation valves would have functioned in the unlikely event of a steam line break and the resulting high steam flow since the valve discs hang at three degrees into the steam flow and have closed on several instances in the past at 4×10^6 lbs./hr. (6.6×10^6 lbs./hr. corresponds to 100% power) steam flow even without receiving a close signal. Therefore, it is concluded that the valve would have performed its safety function had it been required to do so.

Recognizing the repetitiveness of this type of valve failure to fully close at zero or low flow condition without mechanical assistance or the slacking off of the gland packing, plant staff involvement in investigation of solutions to this problem will continue.

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