NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: V A N A S 1 ②	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③	LICENSE TYPE: 4 1 1 1 1 ④	CAT ⑤

CON'T

0 1	REPORT SOURCE: L ⑥	DOCKET NUMBER: 0 5 0 0 0 3 3 8 ⑦	EVENT DATE: 0 9 1 3 7 9 ⑧	REPORT DATE: 1 0 1 2 7 9 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 On September 13, 1979, during Mode 1 operation, loop B steam flow indicator FI-1485

0 3 read lower than the redundant indicator FI-1484. After ensuring that an actual low

0 4 steam flow did not exist, the channel was placed in the tripped condition and

0 5 declared inoperable. Since the affected channel was placed in "Test" and the redun-

0 6 dant channel remained operable, the health and safety of the general public were not

0 7 jeopardized. Reportable pursuant to T.S. 6.9.1.9.b.

0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	I B ⑪	E ⑫	X ⑬	V A L V E X ⑭	F ⑮	D ⑯

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 9	—	1 1 5	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS ㉒	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X ⑱	B ⑲	Z ⑳	Z ㉑	0 0 0 0	Y ㉓	N ㉔	A ㉕	V 1 3 5 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 Steam flow indicator FI-1485 indicated low because the root valves for the trans-

1 1 mitter were leaking causing a loss of reference leg level. The steam flow channel

1 2 was placed in the tripped condition and the loop was calibrated and functionally

1 3 tested satisfactorily. The transmitter root valve will be reworked during the

1 4 current refueling outage.

	FACILITY STATUS	% POWER	OTHER STATUS ㉚	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION ㉜
1 5	E ㉘	0 4 9 ㉙	NA	A ㉛	Operator Observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY ㉟	LOCATION OF RELEASE ㊱
1 6	Z ㉝	Z ㉞	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION ㊴
1 7	0 0 0 ㊲	Z ㊳	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION ㊶
1 8	0 0 0 ㊵	NA

LOSS OF OR DAMAGE TO FACILITY ㊸

	TYPE	DESCRIPTION
1 9	Z ㊷	NA

1168 146

PUBLICITY

	ISSUED	DESCRIPTION ㊺	NRC USE ONLY
2 0	N ㊹	NA	

7910160 491

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Virginia Electric and Power Company
North Anna Power Station, Unit #1
Docket No. 50-338
Report No. LER 79-115/03L-0

Description of Event

On September 13, 1979, during Mode 1 operation at 49% power, loop B steam flow indicator FI-1485 gave a low flow indication. The redundant loop B steam flow indicator FI-1484 did not indicate a low flow. The affected channel was placed in the tripped condition and declared inoperable.

Probable Consequences of Occurrence

Steam line flow is used to initiate both a reactor trip and a safety injection. A safety injection will occur on a high steam line flow coincident with low steam line pressure or low-low TAVG. A reactor trip will be actuated by a steam flow/feedwater flow mismatch and low steam generator level. Since the affected steam flow channel was placed in the tripped condition and the redundant channel remained operable, the health and safety of the general public were not affected by this event. There are no generic implications associated with this occurrence.

Cause of Occurrence

Loop B steam flow indicator FI-1485 gave low indication because the root valves for the flow transmitter were both leaking causing a loss of reference leg level.

Immediate Corrective Action

The channel was placed in the tripped condition and the root valve was temporarily repaired.

Scheduled Corrective Action

The transmitter root valves will be reworked during the refueling outage presently in progress.

Actions Taken to Prevent Recurrence

No further actions are required.