

## UNITED STATES NUCLEAR REGULATORY COMMISSION

REGION IV 611 RYAN PLAZA DRIVE, SUITE 1000 ARLINGTON, TEXAS 76012 CENTRAL FILES
PDR:HQ
LPDR
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NSIC

September 14, 1979

Docket No. 50-285

Omaha Public Power District
ATIN: W. C. Jones, Division Manager Production Operations
1623 Harney Street
Omaha, Nebraska 68102

Gentlemen:

The enclosed IE Information Notice provides information with regard to a potential unreviewed safety question involving high energy lines and control systems.

68102

Sincerely,

Karl V. Seyfrit

Director

#### Enclosures:

1. IE Information Notice 79-22

2. List of IE Information Notices Issued in the Last Six Months

cc: S. C. Stevens, Manager Fort Calhoun Station Post Office Box 98 Fort Calhoun, Nebraska

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# UNITED STATES NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT WASHINGTON, D.C. 20555

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### QUALIFICATION OF CONTROL SYSTEMS

Public Service Electric and Gas Company notified the NRC of a potential unreviewed safety question at their Salem Unit 1 facility. This notification was based in a continuing review by Westinghouse of the environmental qualifications of equipment that they supply for nuclear steam supply systems. Based on the present status of this effort, Westinghouse has informed their customers that the performance of non-safety grade equipment subjected to an adverse environment could impact the protective functions performed by safety grade equipment. These non-safety grade systems include:

Steam generator power operated relief valve control system

Pressurizer power operated relief valve control system

Main feedwater control system

Automatic rod control system

These systems could potentially malfunction due to a high energy line break inside or social of containment. NRC is also concerned that the adverse environment could also give erroneous information to the plant operators. Westinghouse states that the consequences of such an event could possibly be more limiting than results presented in Safety Analysis Reports, however, Westinghouse also states that the severity of the results can be limited by operator actions together with operating characteristics of the safety systems. Further, Westinghouse has recommended to their customers that they review their systems to determine whether any unreviewed safety questions exist.

This IE Information Notice is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If NRC evaluations so indicate, further licensee actions may be requested or required. If you have questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

No written response to this Information Notice is required.

## LISTING OF IE INFORMATION NOTICES ISSUED IN 1979

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Subject	Date Issued	Issued To
Nonconforming Pipe Support Struts	4/16/79	All power reactor facilities with a Construction Permit (CP)
Lower Reactor Vessel Head Insulation Support Problem	5/7/79	All Holders of Reactor Operating Licenses (OLs) Construction Permits (CPs)
Attempted Damage to New Fuel Assemblies	5/11/79	All fuel facilities, research reactors, and power reactors with an Operating Licensee (OL) or a Construction Permit (CP)
Indication of Low Water Level in the Oyster Creek Reactor	5/29/79	All Holders of Operating License (OL) or Construction Permit (CP)
NRC Position of Electrical Cable Support Systems	6/11/79	All Power Reactor facilities with a Construction Permit (CP) and applicants
Deficient Procedures	6/7/79	All Holders of Reactor Operating Licenses (OLs) and Construction Permits (CPs)
Nuclear Incident at Three Mile Island	6/22/79	All Research Reactors and Test Reactors with Operating Licenses (OLs)
Source Holder Assembly Damage From Misfit Between Assembly and Reactor Upper Grid Plate	6/20/79	All Holders of Reactor Operating Licenses (OLs) and Construction Permits (CPs)
	Nonconforming Pipe Support Struts  Lower Reactor Vessel Head Insulation Support Problem  Attempted Damage to New Fuel Assemblies  Indication of Low Water Level in the Oyster Creek Reactor  NRC Position of Electrical Cable Support Systems  Deficient Procedures  Nuclear Incident at Three Mile Island  Source Holder Assembly Damage From Misfit Between Assembly and Reactor Upper	Nonconforming Pipe Support Struts  Lower Reactor Vessel Head Insulation Support Problem  Attempted Damage to New Fuel Assemblies  Indication of Low Water Level in the Oyster Creek Reactor  NRC Position of Electrical Cable Support Systems  Deficient Procedures  6/7/79  Nuclear Incident at Three Mile Island  Source Holder Assembly Damage From Misfit Between Assembly and Reactor Upper

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79-18	Skylab Reentry	7/5/79	All holders of Reactor Operating Licenses (OLs)
79-19	Pipe Cracks in Stagnant Borated Water Systems at PWR Plants	7/17/79	All Holders of Reactor Operating Licenses (OLs) and Construction Permits (CPs)
79-20	NRC Enforcement Policy NRC Licensed Individuals	8/14/79	All Holders of Reactor Operating Licenses (OLs) and Construction Permits (CPs), Research Reactors, Test Reactors Licensees with Licensed Operators
79-21	Transportation and Commerical Burial of Radioactive Material	9/7/79	All power and research reactors with Operating License (OL)

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