

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NYJAFI 200-00000-0000 341111 45
7 8 9 14 15 25 26 30 37 38

CON'T
01 REPORT SOURCE L 6015000333 7090379 8100179 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Please See Attachment
03
04
05
06
07
08

09 SYSTEM CODE SF 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE PUMPPXX 14 COMP. SUBCODE B 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 18 19 20
17 LER NO. REPORT NUMBER 719 21 EVENT YEAR 22 SHUTDOWN METHOD Z 23 SEQUENTIAL REPORT NO. 058 24 OCCURRENCE CODE 03 25 REPORT TYPE L 26 REVISION NO. 0 27
ACTION TAKEN X 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 HOURS 000 22 ATTACHMENT SUBMITTED Y 23 NRPD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER T147 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESC. ON AND CORRECTIVE ACTIONS (27)

10 Please See Attachment
11
12
13
14

15 FACILITY STATUS X 28 % POWER 000 29 OTHER STATUS NA 30 METHOD OF DISCOVERY C 31 DISCOVERY DESCRIPTION See Attachment 32
7 8 9 10 12 13 44 45 46 80
16 ACTIVITY CONTENT Z 33 RELEASED OF RELEASE Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE 36
7 8 9 10 11 44 45 80
17 PERSONNEL EXPOSURES NUMBER 000 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 11 12 13 80
18 PERSONNEL INJURIES NUMBER 000 40 DESCRIPTION NA 41
7 8 9 11 12 80
19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 80
20 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 80

NAME OF PREPARER W. Verne Childs PHONE 315-342-3840
1108 206 NRC USE ONLY

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-058/03L-0

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During plant heat up, as a part of normal startup operations, the HPCI System turbine was intentionally uncoupled from the pump to allow over speed testing of the turbine in accordance with F-ST-4K titled "HPCI Turbine Over Speed Testing." This action, while required by the Insurance Underwriters, caused the HPCI System to be placed in a degraded mode identified in Technical Specifications Appendix A, Paragraph 3.5.C.1.a.

Since the HPCI System is designed for the protection in the event of certain small break LOCA conditions between 150 and 1120 psig, the intentional disabling of the system at a pressure of less than 150 psig, did not represent any significant hazard to the public health and safety.

During the conduct of F-ST-4K, which was satisfactory and was witnessed by a representative of the Insurance Underwriters, reactor pressure was maintained below 150 psig. Following completion of the test the pump and turbine were re-coupled and the system demonstrated operable to meet the requirements of Technical Specifications Appendix A, Paragraph 4.5.C.

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