Report Number:	79-56/01T
Report Date:	9/14/79
Occurrence Date:	8/31/79
Facility:	Salem Generating Station Public Service Electric & Gas Company Hancock's Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

RHR Pump Exceeds Design Runout Flow

CONDITIONS PRIOR TO OCCURRENCE:

Operational Mode 5

DESCRIPTION OF OCCURRENCE:

In response to a NRC question on Salem Unit 2 regarding RHR Pump NPSH during post-LOCA recirculation mode, a Unit 2 RHR Pump was tested for the highest runout flow for the worst hydraulic configuration. This configuration is when one RHR Pump is feeding two charging pumps, two safety injections pumps and also directly into two cold legs. Test results indicated that the RHR Pump flow exceeded the design runout flow. Since the RHR Pump is required to operate under LOCA conditions, the RHR piping configuration existing on Salem Unit 1, which is identical to Unit 2, is unacceptable.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

As indicated by the Unit 2 test results, it would appear that the Unit 1 RHR System flow resistance is low and the pump would runout under the accident mode of operation.

ANALYSIS OF OCCURRENCE:

The Engineering Department evaluation of this condition will be submitted in a supplemental report.

CORRECTIVE ACTION:

A Design Change is in preparation by the Engineering Department which would increase the system flow resistance by resizing the orifices on the flow elements upstream and downstream of the RHR heat exchangers. The Design Change has been accepted by Westinghouse. The estimated time to make this change is approximately three weeks and should be completed during the present outage.

FAILURE DATA:

Not Applicable

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Manager - Salem Generating Station

SORC Meeting No. 70-79

Prepared by A. W. Kapple

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U. S. NUCLEAR REGULATURY COMMISSION NRC FORM 366 (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 0 0 0 0 - 0 0 3 -0 GS1 (2)0 0 -S 0 N LICENSE NUMBER CONT 4 7 9 (9 911 0 2 7 2 7 0 8 3 1 7 9 REPORT 001 0 5 0 1 L (6) REPORT DATE SOURCE EVENT DATE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) response to a NRC question on Salem Unit 2 regarding RHR Pump NPSH during post-0 2 Tn tests performed on Unit 2 RHR System indicated the RHR Pump flow | LOCA operation, 0 3 exceeded the design runout flow. Since Unit 1 has an identical configuration as 0 4 Unit 2, this unacceptable condition exists on Unit 1. Engineering Department is 0 5 evaluating this condition and a supplemental report will be subritted. This is the 0 6 first occurrence of this type. 80 COMP SUBCODE CAUSE CAUSE SYSTEM COMPONENT CODE SUBCODE Z (16 Z ZZI Z (14 13 Z Z Z (12) A F (11 B 0 9 19 13 REVISION REPORT OCCURRENCE SEQUENTIAL NC. YPE CODE REPORT NO. EVENT YEAR LER RO 0 T 1516 0 REPORT 9 7 3 32 NUMBER 28 COMPONENT PRIME COMP SUBMITTED NPRO-4 MANUFAC pes HOURS (22) SHU THOO EFFEC FORMOUB SUPPLIER TIO 9 9 9 9 ZI L 125 (26) N (24) (23) Y 10 01 0 0 Z CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 by test results that the RHR System flow resistance is low. A Design evident is Change to increase flow resistance by resizing the orifices in the flow elements upstream and downstream of the heat exchanger is in preparation. Estimated time complete this change is three weeks. 4 80 q METHOD OF DISCOVERY 32 DISCOVERY DESCRIPTION FACILITY OTHER STATUS S POWER Questio C (31) Test to respond to NRC 010 0 0 29 28 N/A 5 80 45 17 10 ACTIVITY CONTENT LOCATION OF RELEASE 36 (35) AMOUNT OF ACTIVITY RELEASED OF RELEASE N/A 34 N/A (23) Z Z 16 80 PERSONNEL EXPOSURES DESCRIPTION (39 TYPE NUMBER N 38 OR ORIGINA 30 13 NUL IZ PERSONNEL DESCRIPTION (41 NUMBER N/A 40 1.4 80 OF OR DAMAGE TO FACILITY (43) 361344 DESCRIPTION N/A 42 1 9 NRC USE ONLY PUBLICITY DESCRIPTION (45 TIGA 1.1 N/A 63 60 (609) 3 6 A. W. Kapple PHONE -NAME OF PREPARER .