

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(0) (1) [N][Y][J][A][F][1] (2) [0][0]-[0][0][0][0]-[0][0][0][0] (3) [4][1][1][1][1] (4) _____ (5)

CON'T

(0) (1) REPORT SOURCE [X] (6) [0][5][0][0][0][3][3][3] (7) [0][8][3][0][7][9] (8) [0][9][1][2][7][9] (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
Please See Attachment

(0) (2) _____

(0) (3) _____

(0) (4) _____

(0) (5) _____

(0) (6) _____

(0) (7) _____

(0) (8) _____

(0) (9) _____

SYSTEM CODE [S][H] (11) CAUSE CODE [B] (12) CAUSE SUBCODE [C] (13) COMPONENT CODE [Z][Z][Z][Z][Z][Z] (14) COMP. SUBCODE [Z] (15) VALVE SUBCODE [Z] (16)
LER RO REPORT NUMBER [7][9] (17) EVENT YEAR [7][9] (21) SEQUENTIAL REPORT NO. [0][4][6] (24) OCCURRENCE CODE [0][3] (28) REPORT TYPE [T] (30) REVISION NO. [0] (32)
ACTION TAKEN [F] (33) FUTURE ACTION [Z] (34) EFFECT ON PLANT [Z] (35) SHUTDOWN METHOD [Z] (36) HOURS [0][0][0][0] (37) ATTACHMENT SUBMITTED [Y] (41) NPD-4 FORM SUB. [N] (42) PRIME COMP. SUPPLIER [Z] (43) COMPONENT MANUFACTURER [Z][9][9][9] (47)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

(1) (0) _____

Please See Attachment

(1) (1) _____

(1) (2) _____

(1) (3) _____

(1) (4) _____

POOR ORIGINAL

FACILITY STATUS [G] (28) % POWER [0][0][0] (29) OTHER STATUS NA (30) METHOD OF DISCOVERY [D] (31) DISCOVERY DESCRIPTION Notified by A/E (32)

ACTIVITY CONTENT RELEASED OF RELEASE [Z] (33) AMOUNT OF ACTIVITY NA (35) LOCATION OF RELEASE NA (36)

PERSONNEL EXPOSURES NUMBER [0][0][0] (37) TYPE [Z] (38) DESCRIPTION NA (39)

PERSONNEL INJURIES NUMBER [0][0][0] (40) DESCRIPTION NA (41)

LOSS OF OR DAMAGE TO FACILITY TYPE [Z] (42) DESCRIPTION NA (43)

PUBLICITY ISSUED DESCRIPTION NA (45)

960076

7909180 379

NRC USE ONLY

NAME OF PREPARER W. Verne Childs

PHONE: 315-342-3840

FORM NO. NRC-400

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-046/03T-0

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While in a cold shutdown condition and during a study being performed by the Architect Engineer, the Architect Engineer identified deficiencies in the design of the cable separation between the ADS System and the HPCI System. Specifically, the study determined that cable No. 1HP1ARCI03 which is the control cable for the HPCI System Steam Supply inside isolation valve (23-MOV-15) and cable Nos. 1LKDAR020 and 021 which are thermocouple extension cables for the HPCI System Steam Line Leak Detection System were physically routed in some areas within the same cable tray sections as those cables which provide power to those safety relief valves utilized as ADS valves.

Since the ADS and HPCI Systems are considered diverse counterparts and as such are considered as Division I and Division II, respectively, the isolation system for the HPCI Steam Line should be physically separated from the ADS System. The Architect Engineer and FitzPatrick staff have been in contact with the Nuclear Steam Supply System Vendor and it appears that while the cable routing currently used does not appear to disagree with the Nuclear Steam Supply Vendors design specification for Electrical Equipment and Separation of Safe-Guard Systems, the conflict should be eliminated by rerouting the cables in certain areas.

In order for this event to cause a decrease in safe-guard equipment capabilities, the subject cables would be required to undergo a phenomenon known as a "double hot short" or be simultaneously destroyed by a significant fire. In view of this, the event did not represent any significant hazard to the public health and safety.

A plant modification was implemented to reroute portions of the control cable for 23-MOV-15 and the thermocouple extension cables for the HPCI Steam Leak Detection System. This was completed prior to plant startup. The new cable routing was verified to meet the separation requirements for Division I and Division II cables and following installation logic functional tests were conducted with satisfactory results to verify proper installation of the cables.

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