

General Offices: 212 West Michigan Avenue, Jackson, Michigan 49201 • Area Code 517 788-0550 September 11, 1979

Director, Nuclear Reactor Regulation Att Mr Dennis L Ziemann, Chief Operating Reactors Branch No 2 US Nuclear Regulatory Commission Washington, DC 20555

DOCKET 50-155 - LICENSE DPR-6 -BIG ROCK POINT PLANT - RESULTS OF REVIEW OF SAFETY CIRCUIT OVERRIDES

Nuclear Regulatory Commission letter dated November 29, 1978 requested, in part, that Consumers Power Company review safety circuits at Big Rock Point which incorporates a manual override feature. This review was to ensure that overriding any one safety actuation signal did not also cause the bypass of other signals, that physical features were sufficient to facilitate administrative controls, and that use of each override was annunciated at the system level. The results of this review are reported herein.

The only means of manually overriding safety signals at Big Rock Point is a manual bypass feature which interrupts the actuation signal to certain plant valves which would otherwise actuate automatically in response to various signals. This bypass is engaged by lifting up on the control console operating handswitch associated with that valve, causing the control switch to rise approximately one-half inch above its normal position. The control switch remains in this raised position until it is manually pushed to its normal position, thereby disengaging the bypass. This physical change in control switch position enables the operator to observe whether an override is engaged and thus facilitates administrative controls. In no case does actuation of any override prevent the operation of any item of equipment except the valve associated with that handswitch. Positioning of control switches in the "pull to stop" position is controlled by procedures and is used principally for surveillance testing. The valves and handswitches involved are as follows:

Valve No	Valve Description	Control Switch Number
MO-7070/7071	Reactor Emergency Cooling Backup Spray Valves	RMC/5527 and 5528
M0-7064	Reactor Building Emergency Spray Valve	RMC/5514
MO-7051/7061	Reactor Emergency Cooling Spray Valves	RMC/5519 and 5501
MO-7053/7063	Emergency Condenser Outlet Valves	RMC/5508 and 5504

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Annunciation does not presently exist to indicate when one of the above-listed overrides is engaged. Efforts have been initiated to procure and install such annunciation. Both visual and audible monitoring of the operating circuit of each valve listed above will be provided. This annunciation is expected to be installed in late 1980, following the next refueling outage. In the interim, the seven control switches will be appropriately identified to preclude inadvertent override engagement.

David A Bixel (Signed)

David A Bixel Nuclear Licensing Administrator

CC JGKeppler, USNRC

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