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Writer's Direct Dial Number

September 10, 1979
GQL 0444

Director of Nuclear Reactor Regulation
Attn: R.W. Reid, Chief
Operating Reactors Branch No. 4
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Moderator Dilution Accidents

This letter is in response to your letter of February 12, 1979, requesting additional information on TMI-1's potential for a boron dilution accident.

In response to your first concern, the only NaOH valves that require testing are the Engineered Safeguards valves which isolate the NaOH tank from the Decay Heat Removal System (i.e. BS-V2A and BS-V2B). Therefore, Met-Ed will perform an analysis, as defined in your letter of September 14, 1978 to determine the effects of a moderator dilution accident and assess the capability of the operators to respond to this event. This analysis will be forwarded to the NRC no later than October 31, 1979.

In response to your second concern, all of the potential pathways for initiation of a moderator dilution accident have been reviewed. As a result of this review, it has been determined that only one such pathway exists. This is, the boron dilution accident by way of an inadvertant discharge of the NaOH tank into the Decay Heat Removal System while open to the Reactor Coolant System. However, as indicated above, an analysis will be performed to determine the effects of that accident.

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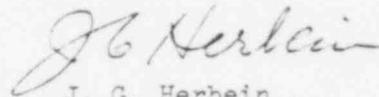
R. W. REID

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Until the time that the results of the analysis are available, Met-Ed will make temporary procedure changes to allow testing of only the sodium hydroxide tank engineered safeguards accuated valve associated with the idle decay heat loop, thus preventing a possible boron dilution accident.

Sincerely,



J. G. Herbein
Vice President
Nuclear Operations

JGH:DGM:mrm

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