

OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE 8/9/79  
 COMPLETED BY C.M. Gaffney  
 TELEPHONE 617-746-7900

OPERATING STATUS

1. Unit Name: Pilgrim I  
 2. Reporting Period: July, 1979  
 3. Licensed Thermal Power (MWt): 1998.  
 4. Nameplate Rating (Gross MWe): 678.  
 5. Design Electrical Rating (Net MWe): 655.  
 6. Maximum Dependable Capacity (Gross MWe): 690.  
 7. Maximum Dependable Capacity (Net MWe): 670.  
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
NONE

Notes

9. Power Level To Which Restricted, If Any (Net MWe): NONE  
 10. Reasons For Restrictions, If Any: N/A.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>5087.0</u>	<u>58223.0</u>
12. Number Of Hours Reactor Was Critical	<u>494.7</u>	<u>4286.6</u>	<u>41334.3</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>465.2</u>	<u>4220.5</u>	<u>39943.1</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>821184.0</u>	<u>8139264.0</u>	<u>66581064.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>279120.0</u>	<u>2794550.0</u>	<u>21925024.0</u>
18. Net Electrical Energy Generated (MWH)	<u>268216.0</u>	<u>2688603.0</u>	<u>21050167.0</u>
19. Unit Service Factor	<u>62.5</u>	<u>83.0</u>	<u>68.6</u>
20. Unit Availability Factor	<u>62.5</u>	<u>83.0</u>	<u>68.6</u>
21. Unit Capacity Factor (Using MDC Net)	<u>53.8</u>	<u>78.9</u>	<u>54.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>55.0</u>	<u>80.7</u>	<u>55.2</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>11.5</u>	<u>10.8</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
NONE

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNIT OPERATING  
 26. Units In Test Status (Prior to Commercial Operation):
- |                      | Forecast     | Achieved     |
|----------------------|--------------|--------------|
| INITIAL CRITICALITY  | <u>_____</u> | <u>_____</u> |
| INITIAL ELECTRICITY  | <u>_____</u> | <u>_____</u> |
| COMMERCIAL OPERATION | <u>_____</u> | <u>_____</u> |

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(1/7)

7908200233

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293  
 UNIT PILGRIM I  
 DATE 8/9/79  
 COMPLETED BY C.M. Gaffney  
 TELEPHONE 617-746-7900

MONTH JULY, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	664.
2	665.
3	664.
4	665.
5	662.
6	662.
7	658.
8	57.
9	240.
10	558.
11	206.
12	0.
13	0.
14	0.
15	0.
16	0.

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	12.
18	459.
19	605.
20	598.
21	213.
22	433.
23	603.
24	589.
25	624.
26	662.
27	28.
28	0
29	0.
30	108.
31	541.

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ORIGINAL  
POCOR

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293  
 UNIT NAME PILGRIM I  
 DATE 8/9/79  
 COMPLETED BY C.M. GAFFNEY  
 TELEPHONE 617-746-7900

REPORT MONTH JULY, 1979

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
6	7/8/79	F	28.2	B	2	79-2/3L-0	CD	VALVEX	LEAK IN HYDRAULIC SYSTEM ON MSIV. LEAK REPAIRED.
7	7/10/79	S	0.0	B	4				MAIN CONDENSER BACKWASH
8	7/11/79	F	153.6	B	2	79-21/1T-0	RB	PIPEXX	LEAK IN CRD RETURN LINE WELD.
9	7/21/79	F	11.5	H	3				REACTOR SCRAMMED LOSS OF VACUUM DURING CONDENSER BACKWASH.
10	7/27/79	F	85.5	B	3	79-27/1T-0	CD	VALVEX	REACTOR SCRAMMED FROM LOSS OF OFF-SITE POWER. MS RELIEF VALVE CYCLED AUTOMATICALLY, VALVES WERE REPLACED.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

(9/77)

811003

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

Month JULY, 1979

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED
29	Check Valve	Leaks	erosion of seat & disc	Renew seat & disc	Disc material changed from ni-resist to S.S.	
29	Service water pump	Excessive vibration	Worn bearings	Renew bearings & pump bowl.	N/A	
3	Piping	Weld Leak	Crack	Renew section of piping.	N/A	
10	Pipe Support	Design Deficiency	Engineering Design	Modified support	N/A	
23	Pipe Support	Design Deficiency	Engineering Design	Modified support	N/A	
10	MOV-1001-36B	Key on valve shaft Sheared			Replaced key and reset limits	
1	"D" Relief Vlv. 'B' Line	Pilot Leakage	Dirt crud	Installed new valves rebuild	Procure new top works	
1	"A" Relief Vlv. 'A' Line	Pilot Leakage	dirt crud	spares - rebuild	Procure new top works	
13	RCIC testable check valve	Leaking packing		Furmanited gland	Repack during future outage.	

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in a letter to Mr. G. C. Andognini dated January 18, 1978.

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket No. 50-293.
2. Scheduled date for next Refueling Shutdown: January 1980.
3. Scheduled date for restart following refueling: April 1980.
- 4.
5. Due to their similarity, requests 4, 5 & 6 are responded to collectively.
6. The fuel, which is presently expected to be loaded during the next scheduled shutdown, may be reloaded fuel of a new design and may therefore require a proposed license submittal and technical specification change. It is not possible, however, to supply pertinent information on dates. As information concerning fuel design, core configuration, Operational review Committee determinations, proposed licensing action, and technical specification submittals become available, it will be forwarded to you.
7. (a) There are 580 fuel assemblies in the core.  
(b) There are 580 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual spent fuel storage capacity is 1170 fuel assemblies and new high density fuel storage racks are in the process of being installed.  
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 580 fuel assemblies (one core).

POOR  
ORIGINAL

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BOSTON EDISON COMPANY

PILGRIM NUCLEAR POWER STATION

Summary of Operations for July, 1979

Unit operated at 100% power until July 8 when power was reduced for a condenser backwash. While the unit was at reduced power timing of the MSIV's was conducted and "D" MSIV failed to meet an acceptable time as per Technical Specifications. A decision was made to take the unit off the line, de-inert the drywell and repair the MSIV. The repair turned out to be a leaking hydraulic line. The unit was returned to service at 0915 on July 9 and while power was being increased, 1-3 water box fouled and a leak was discovered in the CRD return line. A decision was made on July 11 to remove the unit from service and repair the CRD return line. The unit was removed from service at 1132 hrs. on July 11 and repairs were initiated. The repairs were completed and the unit was returned to service at 2110 on July 17. The 1-3 water box remained fouled and a reduction was set to clean the water boxes and the intake piping. During the reduction a scram was experienced from low vacuum at 1313 on July 21. The unit was returned to service at 0035 on July 22. 1-3 water box was still fouled so the condenser was again backwashed on July 24. An unscheduled backwash had to be conducted again on July 25, due to excessive fouling of 1-3 water box. A reactor scram was experienced at 0101 on July 27 due to loss of off-site power caused by a lightning strike. "A" and "D" Relief Valves malfunctioned and had to be replaced. The unit was brought back on line at 1426 on July 30 and obtained 100% output at 2230 hours on July 31.

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