50-320 R. C. Smith

EXTRA

APR 12 1979

MEMORANDUM FOR: Robert B. Minogue, Director
Office of Standards Development

FROM: Guy A. Arlotto, Director
Division of Engineering Standards

SUBJECT: NRC REGULATIONS AND REGULATORY GUIDES

Enclosed, as you instructed at today's meeting, are lists of NRC regulations and Regulatory Guides that are classified as Category "A" and Category "B".

ORIGINAL SIGNED BY G. A. ARLOTTO

Guy A. Arlotto, Director
Division of Engineering Standards
Office of Standards Development

cc: Ray G. Smith, SD:DD

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NRC REGULATIONS

Category A

10 CFR 50.44	Standards for Combustible Gas Control Systems in Light Water Cooled Power Reactors
10 CFR 50.46	Acceptance Criteria for Emergency Core Cooling Systems for Light Water Nuclear Power Reactors
10 CFR 50, Appendix K	ECCS Evaluation Models
10 CFR 55	Operators' Licenses
10 CFR 100	Reactor Site Criteria

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NRC REGULATIONS

Category B

10 CFR 21	Reporting of Defects and Noncompliance
10 CFR 50.55a	Codes and Standards
10 CFR 50, Appendix A	General Design Criteria for Nuclear Power Plants
10 CFR 50, Appendix B	Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants
10 CFR 50, Appendix E	Emergency Plans for Production and Utilization Facilities
10 CFR 50, Appendix G	Fracture Toughness Requirements
10 CFR 50, Appendix H	Reactor Vessel Material Surveillance Program Requirements
10 CFR 50, Appendix J	Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors

Proposed Amendments to Parts 11, 50 and 70 - Clearance Rule

Parts 19 and 20 Control of Radiation Doses to Transient Workers (Proposed)

Parts 20, 30, 40, 50 and 70 Occupational ALARA Rule Change (Proposed)

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REGULATORY GUIDES

Category A

1.3	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Boiling Water Reactors
1.4	Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant Accident for Pressurized Water Reactors
1.5	Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors (Safety Guide 5)
1.7	Control of Combustible Gas Concentrations in Containment Following a Loss-of-Coolant Accident
1.8	Personnel Selection and Training
1.21	Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants
1.23	Onsite Meteorological Programs (Safety Guide 23)
1.33	Quality Assurance Program Requirements (Operation)
1.40	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants
1.47	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems
1.52	Design, Testing, and Maintenance Criteria for Post Accident Engineered-Safety-Feature Atmosphere Cleanup System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants
1.57	Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components
1.73	Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants
1.89	Qualification of Class 1E Equipment for Nuclear Power Plants
1.97	Instrumentation for Light-Water-Cooled Nuclear Power Plants To Assess Plant Conditions During and Following an Accident
1.101	Emergency Planning for Nuclear Power Plants
1.114	Guidance on Being Operator at the Controls of a Nuclear Power Plant
1.131	Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants (For Comment)

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REGULATORY GUIDES

Category A

1.139 Guidance for Residual Heat Removal (For Comment)

1.140 Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants (For Comment)

1.141 Containment Isolation Provisions for Fluid Systems (For Comment)

1.143 Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants (For Comment)

* Criteria for Electric, Instrumentation, and Control Portions of Safety Systems

* Qualification of Electric Modules for Nuclear Power Plants

* Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants

* Recommendation for In Service Testing of Valves which Perform Safety Functions in LWR's.

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* Under Development

REGULATORY GUIDES

Category A

4.1	Programs for Monitoring Radioactivity in the Environs of Nuclear Power Plants
4.6	Measurement of Radionuclides in the Environment - Strontium-89 and Strontium 90 Analyses
4.7	General Site Suitability Criteria for Nuclear Power Stations
*	Bioassay Fission and Corrosion Products

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* Under Development

REGULATORY GUIDES

Category B

1.1	Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps (Safety Guide 1)
1.2	Thermal Shock to Reactor Pressure Vessels (Safety Guide 2)
1.6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems (Safety Guide 6)
1.11	Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11) Supplement to Safety Guide 11, Backfitting Considerations
1.13	Spent Fuel Storage Facility Design Basis (For Comment)
1.16	Reporting of Operating Information--Appendix A Technical Specifications (For Comment)
1.17	Protection of Nuclear Power Plants Against Industrial Sabotage
1.19	Nondestructive Examination of Primary Containment Liner Welds (Safety Guide 19)
1.22	Periodic Testing of Protection System Actuation Functions (Safety Guide 22)
1.24	Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Radioactive Gas Storage Tank Failure (Safety Guide 24)
1.25	Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors (Safety Guide 25)
1.26	Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants (For Comment)
1.28	Quality Assurance Program Requirements (Design and Construction) (For Comment)
1.32	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants
1.45	Reactor Coolant Pressure Boundary Leakage Detection Systems
1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems
1.58	Qualification of Nuclear Power Plant Inspection, Examination, and Testing Personnel

REGULATORY GUIDES

Category B

1.64	Quality Assurance Requirements for the Design of Nuclear Power Plants
1.67	Installation of Overpressure Protection Devices
1.68	Initial Test Programs for Water-Cooled Reactor Power Plants
1.70	Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants
1.75	Physical Independence of Electric Systems
1.79	Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors
1.80	Preoperational Testing of Instrument Air Systems
1.81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants
1.82	Sumps for Emergency Core Cooling and Containment Spray Systems
1.83	Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes
1.88	Collection, Storage, and Maintenance of Nuclear Power Plant Quality Assurance Records
1.90	Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons
1.93	Availability of Electric Power Sources
1.96	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants
1.98	Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor (For Comment)
1.105	Instrument Setpoints
1.106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves
1.107	Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures
1.118	Periodic Testing of Electric Power and Protection Systems
1.120	Fire Protection Guidelines for Nuclear Power Plants (For Comment)
1.121	Bases for Plugging Degraded PWR Steam Generator Tubes (For Comment)
1.136	Material for Concrete Containments
1.142	Safety-Related Concrete Structures for Nuclear Power Plants (Other Than Reactor Vessels and Containments) (For Comment)

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REGULATORY GUIDES

Category ?

DIVISION 1 - POWER REACTORS

DRAFT REGULATORY GUIDES

SC 704-5 Functional Specification for Safety-Related Valve Assemblies in Nuclear Powe.' Plants

DIVISION 1 REGULATORY GUIDES UNDER DEVELOPMENT

POWER REACTORS

* Single-Failure Criteria for Light-Water Reactor Plants Fluid Systems

* Pressurized Water Reactor and Boiling Water Reactor Containment Spray Design Criteria

* Design and Construction Deficiency Reporting Requirements

* Lightning Protection for Nuclear Power Plants

* Application of Torque Switches in Motor Operated Valves Important to Safety

* Guidance for Content of Licensing Applications for Light-Water Reactor Reload Fuel

* Requirements for Qualification Tests and Production Tests for Piping and Equipment Snubbers

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REGULATORY GUIDES

Category B

2.6	Emergency Planning for Research Reactors
4.2	Preparation of Environmental Reports for Nuclear Power Stations
4.8	Environmental Technical Specifications for Nuclear Power Plants
5.54	Standard Format and Content of Safeguards Contingency Plans for Nuclear Power Plants
8.8	Information Relevant to Ensuring that Occupational Radiation Exposure at Nuclear Power Stations will be as low as is Reasonably Achievable
*	Alarming Pocket Dosimeters

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* Under Development