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Vermont Yankee Nuclear Power Station
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Corey R. Daniels
ISFSI Manager

10 CFR 72.48

BVY 19-020

June 20, 2019

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

SUBJECT: 10 CFR 72.48 Report
Vermont Yankee Nuclear Power Station
Docket No. 50-271 and 72-59, 72-1014
License No. DPR-28

Dear Sir or Madam:

In accordance with 10 CFR 72.48(d)(2), Vermont Yankee Nuclear Power Station is required to submit to the NRC a brief description of any changes, tests or experiments made pursuant to 10 CFR 72.48 (c), including a summary of the evaluation of each. Please find attached a summary report which covers the reporting period of March 2, 2017 through March 2, 2019.

This letter contains no new regulatory commitments.

Should you have any questions concerning this letter, or require additional information, please contact Mr. Thomas B. Silko at (802) 451-5354, Ext 2506.

Sincerely,

Steve Naeck

Steven K. Naeck for
CRD/tbs

Attachment: 72.48 Summary Report for the Reporting Period March 2, 2017 through March 2, 2019

IE47
NMSS26
NRR
NMSS

cc: Mr. David C. Lew
Regional Administrator, Region 1
U.S. Nuclear Regulatory Commission
2100 Renaissance Blvd, Suite 100
King of Prussia, PA 19406-2713

Mr. Jack D. Parrott, Senior Project Manager
Office of Nuclear Material Safety and Safeguards
U.S. Nuclear Regulatory Commission
Mail Stop T-5A10
Washington, DC 20555

Ms. June Tierney, Commissioner
Vermont Department of Public Service
112 State Street – Drawer 20
Montpelier, Vermont 05602-2601

Attachment

Vermont Yankee Nuclear Power Station

72.48 Summary Report for the Reporting Period
March 2, 2017 through March 2, 2019

10 CFR 72.48 Summary Report for Reporting Period
March 2, 2017 through March 2, 2019

During the stated reporting period, six 10 CFR 72.48 evaluations were performed at Vermont Yankee (VY). During this period, VY loaded 45 spent fuel casks utilizing the HI-STORM 100 Cask System under CoC No. 72-1014, Amendment 10. This loading campaign completed the transfer of all spent fuel from the VY spent fuel pool to dry cask storage. The six 10 CFR 72.48 evaluations are summarized below. The conclusion for each of these evaluations was that prior NRC approval was not required prior to implantation of the change.

72.48 Screening / Evaluation Number 1277

Description of Change:

The proposed activity accepts the case of a 12" long x 0.2" diameter stainless steel or Inconel wire loaded into a MPC-68 or MPC-68M at Vermont Yankee. Response to Request for Technical Information (RRTI) 2569-012 was issued to provide the technical disposition for accepting the foreign material within the MPC. It was proposed to accept the presence of the foreign material in the MPC as is.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressures are not increased, so no fission product boundary limit is exceeded. No new evaluation methods are used.

72.48 Screening / Evaluation Number 1278

Description of Change:

The proposed activity accepts the case of a 24" long x 0.5" wide x 0.09" thick Nylon tie wrap loaded into a MPC-68 or MPC-68M at Vermont Yankee. RRTI 2569-011 was issued to provide the technical disposition for accepting the foreign material within the MPC. It was proposed to accept the presence of the foreign material in the MPC as is.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressures are not increased, so no fission product boundary limit is exceeded. No new evaluation methods are used.

72.48 Screening / Evaluation Number 1282

Description of Change:

The proposed activity accepts the case of a 5" long x 2" wide x 15 mils thick piece of duct tape loaded into an MPC-68 or MPC-68M at Vermont Yankee. RRTI-2569-010 was issued to provide the technical disposition for accepting the foreign material within the MPC.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM 100 system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressure is not increased beyond acceptable limits, so no fission product boundary limit is exceeded. No new evaluation methods are used.

72.48 Screening / Evaluation Number 1339

Description of Change:

Vermont Yankee (VY) Nuclear Power Plant has elected to use a site-specific fuel debris container that has different design details from the generic BWR Damaged Fuel Container (DFC) described in the HI-STORM 100 FSAR (Figure 2.1.2C in HI-2002444). See Holtec Drawing 11328 for the proposed, VY-specific fuel debris container.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM 100 system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. The Damaged Fuel Container complies with the prescribed requirements. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressures are not increased, so no fission product boundary limit is exceeded. No new evaluation methods are used.

72.48 Screening / Evaluation Number 1340

Description of Change:

Vermont Yankee Nuclear Power Plant has elected to use a site-specific damaged fuel container that has different design details than the generic BWR Damaged Fuel Container (DFC) described in the HI-STORM 100 FSAR (Figure 2.1.2C in HI-2002444). See Holtec Drawing 10617 for the proposed, VY damaged fuel container.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM 100 system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. The Damaged Fuel Container complies with the prescribed requirements. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressures are not increased, so no fission product boundary limit is exceeded. No new evaluation methods are used.

72.48 Screening / Evaluation Number 1352

Description of Change:

For Vermont Yankee HI-STORM 100 Version B casks, radiation and temperature inspections shall be done through the use of a robotic crawler designed to enter the HI-STORM through the Outlet vent. The gamma shield cross plates on the outlet vents shall be temporary removed to allow for this inspection. See RRTI 2569-25 for the proposed inspection plan.

Summary Evaluation of Change:

There are no malfunctions associated with the HI-STORM system due to the proposed activity and so no malfunction likelihood, consequences or results can be increased. The containment boundary remains unchanged, so no accident consequences can be increased. Methods of handling and operating the cask system are not affected, so no new accidents can be created. Cask system temperatures, including fuel cladding, are not increased beyond acceptable limits and MPC internal pressures are not increased, so no fission product boundary limit is exceeded. No new evaluation methods are used.

Note: the Screening / Evaluation numbers are based upon the Holtec International 10 CFR 72.48 Screening /Evaluation Database for Holtec project No. 2569.