

1. Letter from D A Bixel (CP Co) to A Schwencer (NRC), dated February 8, 1977, spent fuel pool modifications, response to Question S6, (2510/0390).
2. VandeWalle, David J, Director, Nuclear Licensing, CP Co, to Director, Nuclear Reactor Regulation, USNRC, "Proposed Technical Specification Change Request - Auxiliary Feedwater System," September 17, 1984.
3. Johnson, B D, CP Co, to Director, Nuclear Reactor Regulation, Attention Mr Dennis M Crutchfield, "Seismic Qualification of Auxiliary Feedwater System," August 19, 1981.
4. Mr Thomas V Wambach (NRC) to K W Berry (CP Co), letter dated July 24, 1987.
5. Facility change, FC-680, Spent Fuel Pool Rerack, (Cartridges 9576 through 9579).
 - a. Safety Analysis Report, "Spent Fuel Storage Modification," October 16, 1986, amended December 19, 1986.
 - b. Westinghouse, "Design Report of Region 2 Spent Fuel Storage Racks Palisades Plant," WNEP-8626, May 1, 1987, (9579/1589).
6. Letter from K W Berry (CP Co) to NRC, dated January 29, 1990, "Response to Generic Letter 89-13, Service Water System Problems Affecting Safety-Related Equipment."
7. Deviation Report Number D-PAL-89-061, "Post-Accident Operation of CCW System," initiated March 23, 1989.
8. Safety Evaluation By the Office of Nuclear Reactor Regulation, Withdrawal of Service Water Temperature Limit, Consumers Power Company, May 4, 1987.
9. Deleted.
10. Engineering Analysis, EA-A-PAL-96-011, "Calculation of Spent Fuel Pool Cooling System Design Heat Load," Rev. 2, April, 2000.
11. Deleted
12. Engineering Analysis, EA-DBD-1.02-003, Rev 1, "West ESG Room Flooding."
13. CPCo Internal Correspondence, DTPerry to GBSzczyпка, "Heat Exchanger (CCW) Flow Limits," dated November 2, 1988.

14. EI Final Report, "Analysis of Palisades Component Cooling and Service Water Systems," Revision 1, dated October 25, 1988 (D045\1075).
15. EA-GCP-93-01, "Review of Current Spent Fuel Pool Criticality Analysis and Bounding Conditions," March 19, 1993, Cart/Frame F474/0631.
16. Facility Change, FC-864, "Dry Storage of Spent Fuel Nuclear Fuel."
17. EA-FC-864-026, "Track Alley Load Distribution System (LDS): Evaluation of Load, Load Combination and Acceptance Criteria," Revision 1.
18. EA-FC-864-30, "FC-864 Heavy Loads," Revision 3.
19. Certificate of Compliance (C of C) for Dry Fuel Storage Casks, No 1007 Revision 0, effective 5/7/93.
20. Deleted
21. Safety Analysis Report for the Ventilated Storage Cask System, prepared by Pacific Sierra Nuclear Associates and Sierra Nuclear Corporation, October 1991.
22. EA-GOTHIC-CST-01, Rev 2, September 2009, "Determination of Initial Condensate Storage Tank (T-2) Indicated Level to Ensure 100,000 Gallons of Available Inventory."
23. EA-Bioshield Temp-01, "Effects of Plant Operating Temperatures on the Palisades Biological Shield Wall," Rev 1, October, 1995.
24. Engineering Change EC 8368, "Replace Existing Control Room HVAC Dampers with Bubble-Tight Dampers," dated June 2007.
25. EA-A-PAL-93-043-01, "NPSH Evaluation for Charging and Boric Acid Pumps," Rev 2, November 2003.
26. Safety Evaluation 94-0288, "Plant Operation With CV-0913 and CV-0950 in the Open Position."
27. EA-C-PAL-95-1299A, "Biological Shield Wall Temperature Profile Based On Measured Temperatures Inside the Reactor Vessel Cavity."
28. Condition Report C-PAL-97-1363, "Component Cooling Water System Not Analyzed for Present LOCA Containment Analysis."
29. EA-LOCA-2001-01, Revision 1, "Containment Response to a LOCA Using CONTEMPT-LT/28."

30. CPCo Letter to NRC, "Steam Binding of AFW Pumps - Response to Generic Letter 88-3," dated May 9, 1988.
31. NRC Correspondence from Robert G. Schaaf to Thomas C. Bordine, Docket No. 50-255, October 28, 1997.
32. Technical Specification Surveillance Procedure RO-216, "Service Water Flow Verification."
33. Internal Memo GFPratt to BNYoung, "A-CMT-96-044 Ultimate Heat Sink Technical Specification Change Request," dated January 9, 1997.
34. CCP Discrepancy Report F-CG-89-058 dated October 31, 1990, "Failure of Nonseismic CCW Piping Inside Containment," (D309/0011).
35. Engineering Analysis EA-GWO7793-01, "CCW Piping Inside Containment HELBA," (C268/1281).
36. EA-JEG-97-01, Revision 0, "Operation of Charging Pumps P-55B & C Without CCW Flow to the Oil Coolers."
37. Letter from NRC to KMHaas, SER for "Change in Commitments to Perform Post-Accident Containment Hydrogen Concentration Analysis and Sump pH Analysis," dated March 27, 1995.
38. Letter from R L Hauter (Consumers) to J F O'Leary (NRC), "Docket No 50-255," dated December 1, 1972 (2504/0226)
39. Letter from R A Vincent (Consumers) to D M Crutchfield (NRC), "Docket No 50-255 - License DPR-20 - Palisades Plant - Systematic Evaluation Program," dated March 31, 1982
40. SC-94-065, "Change Shield Cooling System Flow Rate to 134-154 gpm"
41. EAR-2000-0129, "Replacement of Shield Cooling Pumps"
42. EA-D-PAL-93-272F-01, Revision 1, "Engineered Safeguards Room Heatup Following LOCA in Conjunction with a LOOP" (44782402)
43. Design Basis Document, DBD-7.10, "NFPA 805 Fire Protection Program"
44. NRC Safety Evaluation and Amendment 202 to the Palisades License, June 4, 2001
45. EA-SFP-99-03, Revision 0, "New Fuel Storage Fuel Pool and Fuel Handling Criticality Safety Analysis" (4996/1664)

46. Facility Change FC-976, "Modify L-3 to Single Failure Proof and Increase Capacity from 100 Tons to 110 Tons"
47. EA-D-PAL-88-222B, Revision 0, "Minimum Lake Level Calculation for P-5 Suction With Five Discharge Tubes Open" (F043/1540)
48. Engineering Analysis EA-MOD-2003-019-05, "Evaluation of Heavy Loads for NUHOMS System," Revision 0
49. Standardized NUHOMS® Horizontal Modular Storage System Certificate of Compliance (C of C) Number 1004, Amendment No. 5
50. Stang, John F., Senior Project Manager, Project Directorate III-1, USNRC, to Daniel J. Malone, Palisades Nuclear Plant Site President, "Amendment No. 215 to the Palisades Operating Licensee No DPR-20: Palisades Spent Fuel Pool Crane Upgrade (TAC No. MC1862)," June 16, 2004
51. USNRC Generic Letter 96-06, Assurance of Equipment Operability and Containment Integrity During Design Basis Accident Conditions," September 30, 1996
52. EA-GL-96-06-SWS-02, "Service Water GL-96-06 Waterhammer Assessment," August 12, 2004
53. Generic Letter 96-06, Waterhammer Issues Resolution, Technical Basis Report, EPRI, Final Report April 2002
54. Generic Letter 96-06, Waterhammer Issues Resolution, User's Manual, EPRI, Final Report, April 2002
55. NRC Acceptance Letter of EPRI TR-113594, dated April 3, 2002, and enclosed Safety Evaluation Report, Evaluation of Electric Power Research Institute Report, TR-113594, "Resolution of Generic Letter 96-06 Waterhammer Issues," dated December 2000
56. 10 CFR 50.44, Combustible Gas Control for Nuclear Power Reactors, effective October 16, 2003
57. Engineering Analysis EA-EC-235-01, Revision 0, "Assessment of the High Pressure Air System's Capacity to Cycle Valves in the West Engineering Safeguards," November 2005
58. Bechtel Specification 5395-M-73, "Specification for the Reactor Crane," Revision 1, dated March 8, 1967
59. Letter from Dresser Crane, Hoist & Tower Division to Bechtel Corporation dated September 9, 1970 (Cart./Frame G728/1146)

60. Bechtel Inter-office Memo dated December 18, 1969 (Cart/Frame 0447/2123)
61. Bechtel Field Procedure Number 3, Revision 6, Change #7, Load Test of Up-Rated Polar Crane, 2/22/1970 (Cart./Frame 0183/0181)
62. NRC letter from Dennis M. Crutchfield to David J. VandeWalle, "Control of Heavy Loads (Phase 1) – NUREG-0612," dated November 9, 1983
63. Consumers Power Letter from Gerald B. Slade to the Nuclear Regulatory Commission, "Completion of Actions Made in Response to NRC Bulletin 88-04 Potential Safety Related Pump Loss," dated October 30, 1990.
64. Engineering Change EC516, "CCW System Temperature Re-Rate Design Change," dated January 25, 2007.
65. EA-CA025644-01, Revision 1, Evaluation of the Impact of 110% Emergency Diesel Generator Overload Operating Condition on Ambient Temperature, August 2007 (later).
66. EA-EC9600-01, Revision 1, Functionality of Electrical Equipment in the Emergency Diesel Room at an Elevated Temperature of 121°F, April 2007 (later).
67. EA-WJB-00-01, Revision 2, Spent Fuel Pool Dilution Analysis, EC0044749.
68. Engineering Analysis EA-CAP047706-01, "Palisades Reactor Head Drop Analysis," dated July 3, 2007.
69. AREVA NP Inc., ANP-2858P-003, Palisades SFP Region 1 Criticality Evaluation with Burnup Credit, January 2011.
70. EA-EEQ-TEMP-01, Revision 2, "EEQ Average Arrhenius Temperature Outside Containment Building Based on Temperature Monitoring"
71. EA-BHS-EQ-2000-01, Revision 4, "Results of Temperature Monitoring in Containment and Suggested Methods for Use of the Temperature Data in EQ Qualified Life Calculations"
72. EA-BDM-88-02, Revision 0, "Impact on EEQ Files Due to Increase of Containment Operating Temperature from 138°F to 145°F"
73. EA-EC46993-01, Revision 0, "E-53A/B Spent Fuel Pool Heat Exchanger Tube Plugging Calculation "
74. EA-CCW-87-01, Revision 3, "Spent Fuel Pool Heat Load and Required SFPHX Component Cooling Water Flow During a DBA"
75. EA-EC32159-05, Revision 0, "Criticality Safety Evaluation for Region 1 Spent Fuel Racks at Palisades (HI-2115017)"

76. PLP-RPT-12-00053, "NFPA Code Conformance Review"
77. EA-EC46465-09, Revision 0, "Evaluation of ADV Air Requirements for FLEX Event."
78. EA-EC32159-01, Revision 0, "Bulk Thermal-Hydraulic Analyses for the Palisades SFP and Tilt Pit with New Region 1 Racks (HI-2114999)"
79. Final Safety Analysis Report for the HI-STORM FW MPC Storage System, Holtec Report HI-2114830, Revision 4
80. Certificate of Compliance No. 1032, Amendment 1, Revision 1, for HI-STORM FW MPC Storage System
81. Engineering Change EC 63832, "Add an Air Accumulator to CV-0910 and Appendix J Program Update."
82. Certificate of Compliance for Dry Fuel Storage Casks, Certificate No. 1007 Revision 0 Renewal, effective September 20, 2017.
83. Final Safety Analysis Report for the VSC-24 Ventilated Storage Cask System, prepared by Energy Solutions, Revision 9, October 2017.
84. NRC letter, "Palisades Nuclear Plant - Issuance of Amendment Regarding Transition to Risk-Informed, Performance-Based Fire Protection Program in Accordance with 10 CFR 50.48(c)," dated February 27, 2015 (ADAMS Accession No. ML15007A191).