Vogtle PEmails

From:	Habib, Donald
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То:	Vogtle PEmails
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Background Information As a result of experience associated with construction of Vogtle Units 3&4, NRC issued SECY-19-0034 which refines the general principles for Tier 1 content to focus on safety significance	CY-19-0034 states the Staff is taking steps to improve Tier 1 content that NRC approval is not required for design changes of minimal ety significance Fundamentally, the Staff's new approach promotes safety by focusing licensee and Staff attention on those matters most important to safety Also, resources will not be diverted to regulatory approvals for changes of minimal safety significance Avoiding such inefficiencies is particularly important during the last year of facility	····	
 Bac As a result of experience a 3&4, NRC issued SECY-1 Tier 1 content to focus on 	 SECY-19-0034 states the so that NRC approval is n so that NRC approval is n safety significance Fundamentally, the Staff's n Staff attention on those math staff attention on those math safety significance Also, resources will not be d safety significance Avoiding such inefficiencies 	construction when substanti of ITAAC one ream. ONE VISION. ONE GOAL. Vogtle 3&4 Building the future of nucle	0

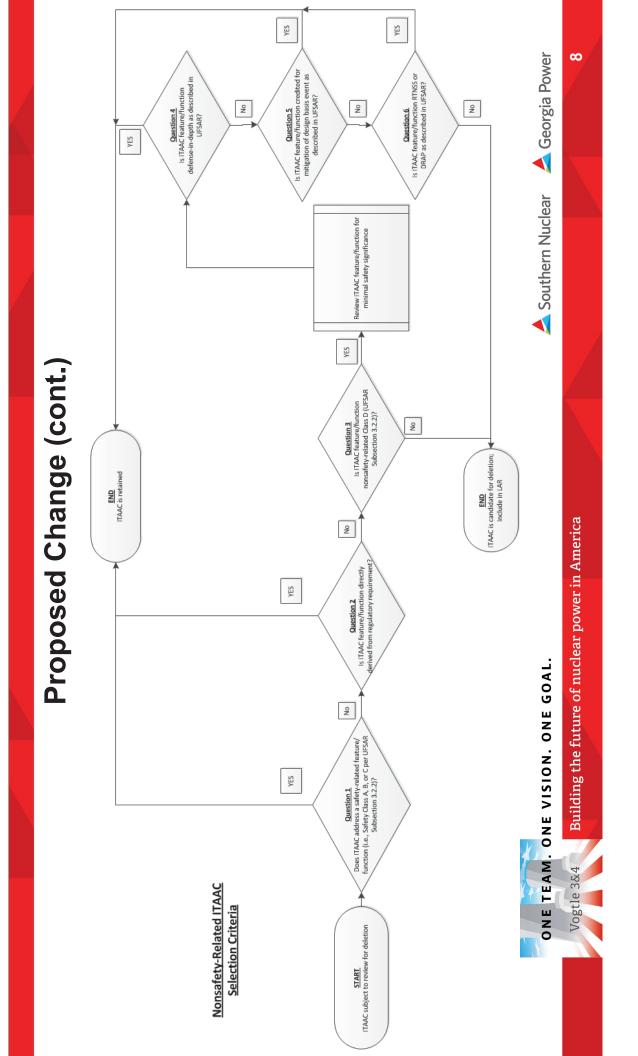
(cont.)	es for Tier 1 content in tely emphasize the 1	s and performance ety	orinciples for Tier 1 content: nd functional level of detail; and, e NRC approval for departures inificance	the Staff will entertain proposals to apply the te-ranging or more limited basis the form of a plant-specific departure from Tier 1	A Southern Nuclear A Georgia Power
Background Information (cont.)	 SECY-19-0034 indicates that the general principles for Tier 1 content in NUREG-0800 (SRP) Section 14.3 do not adequately emphasize the importance of avoiding unnecessary detail in Tier 1 	 Tier 1 should include the top-level design features and performance characteristics that are the most significant to safety 	 To address this, the Staff is refining two general principles for Tier 1 content: 1. Tier 1 should typically be described at a qualitative and functional level of detail; and, 2. Tier 1 should not include detail that could necessitate NRC approval for departures from the certified design that have minimal safety significance 	 SECY-19-0034 indicates that the Staff will entertain proposal new approach on either a wide-ranging or more limited basis Such proposals may come in the form of a plant-specific departure 	ONE TEAM. ONE VISION. ONE GOAL. Vogtle 3&4 Building the future of nuclear power in America

Proposed Change	 SNC is evaluating a potential LAR and Exemption Request to reduce ITAAC with minimal safety significance in accordance with guidance in SECY-19-0034 Preliminary scoping has identified approximately 65 nonsafety-related ITAAC as potential candidates for elimination based on safety significance 	 10 CFR 52.63(b)(1) requires a COL holder referencing a standard design certification to obtain an exemption to depart from the certification information An exemption may be granted only if the exemption will comply with the requirements of §50.12 (as required by §52.7) 	Considering the requirements of §50.12, the requested exemption would be authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security	Special circumstances are present consistent with §50.12(a)(2)(iii)	ONE TEAM. ONE VISION. ONE GOAL. Vogtle 3&4 Building the future of nuclear power in America
	 SNC is ev safety sign Prelimina elimina 	 10 CFR 5 obtain an An ex of §50 	 Consideri law, will n the comm 	 Special ci 	ONE TEAN Vogtle 3&4

	nation	the nonsafety significant	-R 52 App D Section VIII to determine if	n constructed and will be ns of the Atomic Energy Act, nificance of the SSC function	it is a candidate for inclusion	ITAAC SSCs Equipment Class D	📥 Southern Nuclear 🛛 📥 Georgia Power	Q
Proposed Change (cont.)	 Tier 1 will continue to include the most safety-significant information 	 Tier 2 contains detailed design and/or testing requirements for the nonsafety significant ITAAC functions 	 Any change to nonsafety-significant functions will continue to be screened under 10 CFR 52 App D Section VIII to determine if prior NRC approval is required. Changes to UFSAR Chapter 14 test requirements require notification to the NRC 	 This maintains reasonable assurance that the facility has been constructed and will be operated in conformity with the combined license, the provisions of the Atomic Energy Act, and applicable regulations, commensurate with the safety-significance of the SSC function as described in SECY-19-0034 	 ITAAC are reviewed against screening criteria to determine if it is a candidate for inclusion in the LAR 	 Is ITAAC directly derived from regulation? If yes, screens out of LAR What is the AP1000 Equipment Classification of the SSC? Focus on ITAAC SSCs Equipment Class D and lower 	ONE TEAM. ONE VISION. ONE GOAL.	Vogtle 3&4 Building the future of nuclear power in America

	om regulatory	olicable	of the ITAAC is) described in a	Southern Nuclear 📥 Georgia Power	G
ont.)	ctly derived f	ntation for ap	tance Criteria merical value	Southe	
Proposed Change (cont.) Regulatory Requirements	 ITAAC are reviewed to determine if they are directly derived from regulatory requirements 	 Includes review of system design basis documentation for applicable regulations 	 Review of regulation is to determine if the Acceptance Criteria of the ITAAC is needed to satisfy a specific requirement (e.g. numerical value) described in a regulation 	ONE TEAM. ONE VISION. ONE GOAL.	Vogtle 3&4 Building the future of nuclear power in America

) SAR Subsection 3.2.2)	procurement, inspection or e SC-I SCs that are SC-II Jation of passive safety systems JRAP	clear 📥 Georgia Powe	
Proposed Change (cont.) <u>AP1000 Nonsafety Equipment Class Descriptions (UFSAR Subsection 3.2.2)</u>	 Equipment Class D SSCs Class D is nonsafety-related with some additional requirements on procurement, inspection or monitoring ClocFR50 Appendix B and 10CFR21 apply to Class D SSCs that are SC-I Pertinent portions of 10CFR50 Appendix B apply to Class D SSCs that are SC-I Bertinent portions of 10CFR50 Appendix B apply to Class D SSCs that are SC-I Defense-in-depth Some Class D SSCs are considered risk significant as defined by DRAP Equipment Class E and lower SSCs 	 Class E and lower are nonsafety-related; no safety-related function to perform No quality assurance requirements 10CFR50 Appendix B and 10CFR21 do not apply No other regulatory guidance requiring SSC inclusion in Equipment Class D or higher ONE TEAM. ONE VISION. ONE GOAL. 	vogue 3x4 building the future of nuclear power in America



Example 1 - Security ITAAC	C 860 inspects the illumination in the isolation zones and external areas of the a	Security System is Equipment Class E	eptance Criteria for ITAAC 860 states: The illumination in isolation zones and exterior areas within the protected area is 0.2 foot candles measured horizontally at ground level or, alternatively, sufficient to permit observation.	 ITAAC requirements are directly derived from 10 CFR 73.55: The licensee shall provide a minimum illumination level of 0.2 foot-candles, measured horizontally at ground level, in the isolation zones and appropriate exterior areas within the protected area. 	ITAAC 860 screens out of scope of the LAR because Acceptance Criteria is directly derived from regulation Specific numerical value of 0.2 foot candles 	ve vision. One GOAL. 📥 Southern Nuclear 👌 Georgia Power	Vogtle 3&4 Building the future of nuclear power in America
Exa	 Security ITAAC 860 inspects the ill Protected Area 	 Security System is Equip 	 Acceptance Criteria for ITAAC 860 The illumination in isolation zones ar measured horizontally at ground leve 	 ITAAC requirements are The licensee shall proviground level, in the isola 	 ITAAC 860 screens out from regulation Specific numerical value 	ONE TEAM. ONE VISION. ONE GOAL.	Vogtle 3&4 Building the fu

Proposed Change (cont.) Example 2 – Communication Systems (EFS) ITAAC	2.3.19 Communication System	Design Description The communication system (EFS) provides intraplant communications during normal, maintenance, transient, fire, and accident conditions, including loss of offsite power.	 a) The EFS has handsets, amplifiers, loudspeakers, and siren tone generators connected as a telephone/page system. 	ONE TEAM. ONE VISION. ONE GOAL. Vogtle 364 Building the future of nuclear power in America
<u>Example 2 – Con</u>	2.3.19 Communica	Design Description The communication similar maintenance, transier	 a) The EFS has l as a telephone 	ONE TEAM. ONE VISIOI Vogtle 3&4 Building th

Example 2 (cont.) – Communication Systems (EFS) ITAAC: Proposed Change (cont.)

NRC ITAAC #
2.3.19.02a



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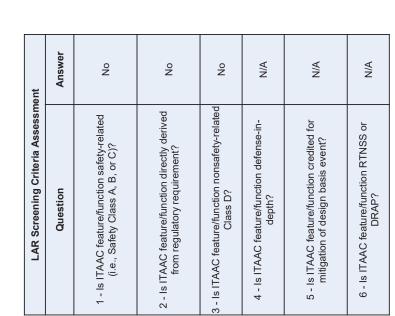
Proposed Change (cont.)

Example 2 (cont.) – Communication Systems (EFS) ITAAC:

- EFS is Equipment Class E (nonsafety-related)
- Review of regulation (10 CFR 73.45) found ITAAC requirements NOT directly derived from regulation
- Regulation has general requirement for communication systems but no specific design detail the ITAAC would directly satisfy
- ITAAC is not defense-in-depth, credited for mitigation of a design basis event or RTNSS/DRAP component
- ITAAC 486 screens into the scope of the LAR

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n t.) (EFS) ITAAC:	s no safety-related function and prises conventional types of le operation routine use will demonstrate luring emergency situations, are lity when required es to required offsite locations in the Emergency Plan n 14.2.9.4.13	 Southern Nuclear Georgia Power 13
Proposed Change (cont.) Example 2 (cont.) – Communication Systems (EFS) ITAAC:	 Technical Justification Aspects Include: As described in UFSAR Subsection 9.5.2, EFS serves no safety-related function and therefore has no nuclear safety design basis As described in UFSAR Subsection 9.5.2.4, EFS comprises conventional types of communication systems which have a history of reliable operation Most of these systems are in routine use, and this routine use will demonstrate their availability Those systems not frequently used, but required during emergency situations, are tested at periodic intervals to demonstrate operability when required As described in UFSAR Subsection 9.5.2.5.1, Interfaces to required offsite locations and emergency offsite communications is addressed in the Emergency Plan System is tested as part of ITP per UFSAR Subsection 14.2.9.4.13 	ONE TEAM. ONE VISION. ONE GOAL. Vogtle 3&4 Building the future of nuclear power in America

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Proposed Change (cont.) Example 3 – Gaseous Radwaste System (WGS) ITAAC:	Gaseous Radwaste System	Design Description	The gaseous radwaste system (WGS) receives, processes, and discharges the radioactive waste gases received within acceptable off-site release limits during normal modes of plant operation including power generation, shutdown and refueling.	The WGS is as shown in Figure 2.3.11-1 and the component locations of the WGS are as shown in Table 2.3.11-3.	The functional arrangement of the WGS is as described in the Design Description of this Section 2.3.11.	The equipment identified in Table 2.3.11-1 can withstand the appropriate seismic design basis loads without loss of its structural integrity function.	The WGS provides the nonsafety-related functions of:	Processing radioactive gases prior to discharge.	b. Controlling the releases of radioactive materials in gaseous effluents.	The WGS is purged with nitrogen on indication of high oxygen levels in the system.	DN. ONE GOAL.	Building the future of nuclear power in America
Example 3 – Ga	2.3.11	Design D	The gase waste gas operation	The WGS shown in	1. The fu Sectio	2. The e basis	3. The M	a. Pr	ы Ч	C. Th	ONE TEAM. ONE VISION. ONE GOAL.	Vogtle 3&4 Building

Example 3 (cont.) – Gaseous Radwaste System (WGS) ITAAC: Proposed Change (cont.)

s the PL-V051 to							
Acceptance Criteria	n valve WGS-I						
Acceptan	A simulated high radiation signal causes the discharge control isolation valve WGS-PL-V051 to close.						
	A simulate discharge close.						
ITA	Tests will be performed to confirm that the presence of a simulated high radiation A simul signal from the discharge discharge radiation monitor, WGS-017, close. causes the discharge control isolation valve WGS- PL-V051 to close.						
Design Commitment	3.b) The WGS provides the nonsafety- related function of controlling the releases of radioactive materials in gaseous effluents. Tests will be performed to confirm that the presence of a simulated high radiation radiation monitor, WGS-01' causes the discharge control isolation valve WGS PL-V051 to close.						
ITAAC #	2.3.11.03b						
System NRC Index #	WGS 454						
System							



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Proposed Change (cont.)

- Example 3 (cont.) Gaseous Radwaste System (WGS) ITAAC:
- WGS V051 is Equipment Class D (nonsafetyrelated)
- ITAAC is not directly derived from regulatory requirements
- Reviewed against 10 CFR 20.1406
- No direct requirements for the V051 function, description of the component or numerical values
- WGS V051 is not a defense-in-depth component, not credited for mitigation of design basis event, and is not a RTNSS/DRAP component
- WGS ITAAC 454 screens in to the scope of the LAR •



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Answer Yes å Ŷ å ٩ ۶ LAR Screening Criteria Assessment Is ITAAC feature/function nonsafety-related Class D? 2 - Is ITAAC feature/function directly derived from regulatory requirement? 1 - Is ITAAC feature/function safety-related 5 - Is ITAAC feature/function credited for mitigation of design basis event? 4 - Is ITAAC feature/function defense-in-depth? 6 - Is ITAAC feature/function RTNSS or DRAP? (i.e., Safety Class A, B, or C)? Question

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ont.) tem (WGS) ITAAC:	serves no safety-related function	the preoperational tests include automatic closure of the ve, WGS-PL-V051, upon receipt of a simulated high	in UFSAR Subsection 14.2.9.3.2	A Southern Nuclear A Georgia Power
Proposed Change (cont.) <u>Example 3 (cont.) – Gaseous Radwaste System (WGS) ITAAC:</u>	 Technical Justification Aspects Include: As described in UFSAR Subsection 11.3.1.1, WGS serves no safety-related function and therefore has no nuclear safety design basis WGS is not a safety significant system 	 Per UFSAR Section 11.3.4.1, the preoperational tests include automatic closure of discharge control/isolation valve, WGS-PL-V051, upon receipt of a simulated high radiation signal 	 WGS V051 is tested as part of the ITP as described in UFSAR Subsection 14.2.9.3.2 	ONE TEAM. ONE VISION. ONE GOAL. Vogtle 3&4 Building the future of nuclear power in America



- Next Steps:
 July 11—Draft LAR to NRC
 July 25—Pre-submittal meeting
 July 31—Submit LAR



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Questions?