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MAY 2 3 2019

U. S. Nuclear Regulatory Commission

Attn: Document Control Desk Washington, DC 20555-0001

10 CFR 50.73

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 50-388/2019-001-00 UNIT 2 LICENSE NO. NPF-22 PLA-7794

**Docket No. 50-388** 

Attached is Licensee Event Report (LER) 50-388/2019-001-00. This LER is reporting an event involving inoperability of a Main Steam Line Isolation Valve (MSIV) that was determined to be reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

There were no actual consequences to the health and safety of the public as a result of this event.

This letter contains no new regulatory commitments.

K. Cimorelli

Attachment: LER 50-388/2019-001-00

Copy: NRC Region I

Ms. T. E. Hood, NRC Project Manager

Ms. L. H. Micewski, NRC Sr. Resident Inspector

Mr. M. Shields, PA DEP/BRP

Ms. J. C. Tobin, NRC Project Manager

#### NRC FORM 366 (04-2018)

#### U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: N	O. 3150-010	)4
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EXPIRES: 03/31/2020



### LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)
(See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

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1. Facility Name Susquehanna Steam Electric Station Unit 2							Docket Number 3. Pag			3. Page	=					
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4. Title Main S	Steam	Isolatio	n Valve	e Leakag	ge											
5. Event Date 6. LER Number 7. Report Date						ate	8. Other Facilities Involved					d				
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5			20.2201(b) 20.2203(a)(3)(i)						☐ 50.73(a)(2)(ii)(A)				☐ 50.73(a)(2)(viii)(A)			
	•		☐ 20.2201(d) ☐ 20.2203(a)(3)(ii)						☐ 50.73(a)(i	☐ 50.73(a)(2)(viii)(B)						
Γ			☐ 20.2203(a)(1) ☐ 20.2203(a)(4)						☐ 50.73(a)(2)(iii)			☐ 50.73(a)(2)(ix)(A)				
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10. Power Level			20.2203(a)(2)(ii) 50.36(c)(1)(ii)(A)						☐ 50.73(a)(2)(v)(A)			73.7	☐ 73.71(a)(4)			
			☐ 20.2203(a)(2)(iii) ☐ 50.36(c)(2)						☐ 50.73(a)(2)(v)(B) ☐			73.7	☐ 73.71(a)(5)			
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			☐ 20.2203(a)(2)(vi) ☐ 50.73(a)(2)(i)(B)						☐ 50.73(a)(2)(vii)			73.7	73.77(a)(2)(iii)			
			□ 50.73(a)(2)(i)(C)						☐ Other (Specify in Abstract below or in NRC Form				n 366	A		
						12. Lice	nsee Co	ontact for t	his LER					obsessed and a second		
icensee Contact C. E. Manges, Jr., Senior Engineer – Nuclear Regulatory Affa							airs	Telephone Number (Include Area Code) 570-542-3089				ode)				
				13. Ce	omplete C	ne Line for e	each Con	nponent Fai	lure Described i	n this Rep	ort					
Cause		System	Compo	onent Ma	nufacturer	Reportable To IC	To ICES	Cause	System	Com	Component		rer	Reportable To ICE		
14. Supplemental Report Expected											Month	Da	y	Year		
Yes (If yes, complete 15. Expected Submission Date) No						15. Expected Submission Date 06			06	14	.	2019				
Abstract (I	imit to 14	00 spaces, i.	e., approxi	mately 14 sing	gle-spaced	typewritten lir	nes)									
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During the Susquehanna Unit 2 refueling outage, Local Leak Rate Testing (LLRT) conducted on March 27, 2019 determined combined as-found leakage through the inboard Main Steam Isolation Valve (MSIV) (HV241F022D) and the outboard MSIV (HV241F028D) in the "D" Main Steam Line was 55,425 standard cubic centimeters per minute (sccm). Subsequent testing, on March 28, 2019 at approximately 20:30, measured the leakage through HV241F028D as 47,791 sccm, which exceeded the Technical Specification (TS) Surveillance Requirement (SR) 3.6.1.3.12 limit of 100 standard cubic feet per hour (scfh) (47,194 sccm) for individual valve leakage.

Based on preliminary cause and history, there is firm evidence that the condition existed during the last operating cycle for longer than allowed by TS 3.6.1.3. The condition is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

Cause information will be provided in a supplement to this LER. Repairs made to the valve resulted in an acceptable as-left LLRT.

There were no actual safety consequences associated with the condition.

NRC FORM 366A (04-2018)

#### U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

EXPIRES: 03/31/2020



# LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

(See NUREG-1022, R.3 for instruction and guidance for completing this form <a href="http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/">http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/</a>)

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1. FACILITY NAME	2. DOCKET NUMBER		3. LER NUMBER				
Susquehanna Steam Electric Station, Unit 2	05000-388	YEAR	SEQUENTIAL NUMBER	REV NO.			
		2019	001	00			

NARRATIVE

# CONDITIONS PRIOR TO EVENT

Unit 1 – Mode 1, approximately 100 percent Rated Thermal Power

Unit 2 - Mode 5, 0 percent Rated Thermal Power

There were no structures, systems, or components that were inoperable at the start of the event that contributed to the event.

#### **EVENT DESCRIPTION**

During the Susquehanna Unit 2 refueling outage, an LLRT conducted on March 27, 2019 determined combined as-found leakage through the inboard MSIV (HV241F022D) [EIIS System / Component Code: SB/ISV] and the outboard MSIV (HV241F028D) [EIIS System / Component Code: SB/ISV] in the "D" Main Steam Line was 55,425 sccm. Subsequent testing, on March 28, 2019 at approximately 20:30, measured the leakage through HV241F028D as 47,791 sccm, which exceeded the TS SR 3.6.1.3.12 limit of 100 scfh (47,194 sccm) for individual valve leakage.

The valve was repaired and an as-left LLRT measured a total penetration leakage value of 28,201 sccm.

Based on preliminary cause and history, there is firm evidence that the condition existed during the last operating cycle for longer than allowed by TS SR 3.6.1.3.12 The condition is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

## CAUSE OF EVENT

The cause of the condition will be provided in a supplement to this LER.

## ANALYSIS/SAFETY SIGNIFICANCE

The redundant MSIV (HV241F022D) in the "D" Main Steam Line had an as found individual leakage value of 7634 sccm (approximately 16.2 scfh) which is below the TS SR 3.6.1.3.12 limit of 100 scfh (47,194 sccm). As a result, there was no loss of safety function for the redundant valve. Additionally, engineering evaluation of the condition demonstrates that the dose consequences remain within the regulatory limit of 5 rem Total Effective Dose Equivalent (TEDE) for the control room and 25 rem TEDE for the low population zone (LPZ) and exclusion area boundary (EAB). As such, for the identified condition, there was no loss of safety function for the main steam penetration nor any actual or potential consequences to the health and safety of the public.

## CORRECTIVE ACTIONS

Corrective actions included repairs to the valve resulting in an acceptable as-left LLRT.

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NRC FORM 366A

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Susquehanna Steam Electric Station, Unit 2	05000-388	YEAR	SEQUENTIAL NUMBER	REV NO.		
		2019	001	00		

## COMPONENT FAILURE INFORMATION

Component Identification - HV241F028D

Component Name - Unit 2 Main Steam Line 'D' Outboard Isolation Valve

Valve Manufacturer - Atwood & Morrill (Acquired By Weir Valve)

Valve Type - Wye Globe

Valve Size - 26 inch

Actuator Manufacturer - Hanna

Actuator Type – Tandem Cylinder

### PREVIOUS OCCURENCES

The following are the most recent Susquehanna LERs involving MSIV leakage:

LER 50-387/2018-003-01, Main Steam Isolation Valve Leakage Due to Pilot Poppet and Pilot Poppet Seat Wear/Degradation, dated September 5, 2018

LER 50-387/2012-006-01, "D" Outboard Main Steam Isolation Valve Leakage - 2012, dated August 8, 2013

LER 50-387/2010-004-00, "D" Outboard Main Steam Isolation Valve Leakage - 2010, dated August 8, 2013

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