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TO: Edson G. Case

FROM: Virginia Elec & Power Co  
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Supplemental Information to Amdt to OL  
Tech Specs Change No. 57.....trans:

PLANT NAME: Surry 1 & 2

retype Per LML 8-31-77

ENCLOSURE

Report, "Large Break LOCA-ECCS Reanalysis  
dtd 8-26-77

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DISTRIBUTION AFTER ISSUANCE OF OPERATING LICENSE

NRC FORM 195  
(2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER  
**50-280/281**  
FILE NUMBER

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO:  
Mr. Edson G. Case

FROM:  
Virginia Electric & Pwr. Company  
G. M. Stallings/Richmond, Va.

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**ACKNOWLEDGED**

PLANT NAME: Surry Units 1 & 2 (2-P)

RJL 8/29/77

ENCLOSURE

Report Entitled:  
Large Break LOCA-ECCS  
Reanalysis

Surry Power Station  
Units No. 1 and 2

Notorized August 26, 1977.....with  
associated changes to tech specs No. 57  
for license No. DPR-32 and DPR-37.....

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VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

August 26, 1977

Mr. Edson G. Case, Acting Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. R. W. Reid, Chief  
Operating Reactors Branch No. 4  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Serial No. 372  
FR/MLB/RWC/EJL:krb  
Docket Nos. 50-280  
50-281  
License Nos. DPR-32  
DPR-37

Dear Mr. Case:

Regulatory

File 37

*pdh*

SUPPLEMENTAL INFORMATION TO AMENDMENT TO THE  
OPERATING LICENSE TECHNICAL SPECIFICATIONS CHANGE NO. 57  
SURRY POWER STATION UNITS 1 AND 2

Pursuant to 10 CFR 50.59, the Virginia Electric and Power Company, in our letter of August 9, 1977 (Serial No. 344), requested an amendment to Operating Licenses DPR-32 and DPR-37 for the Surry Power Station, Units No. 1 and 2. The amendment requested was a change to the Technical Specifications, designated as Change No. 57, which was based on an evaluation of the impact of higher steam generator tube plugging levels on the currently applicable non-LOCA safety analyses. However, the amendment did not include any Technical Specifications changes or reanalysis associated with the impact of higher steam generator tube plugging on LOCA-ECCS performance. This was to be sent in as supplemental information at a later date.

Accordingly, transmitted herewith as Attachments 1 and 2 are (1) a reanalysis of the impact of higher steam generator tube plugging, up to a level of 25%, on the limiting break LOCA-ECCS case ( $C_D=0.4$ ), and (2) the associated changes to the Technical Specifications (including, for completeness the changes already transmitted in the August 9, 1977 amendment). The results of the normally non-limiting break cases ( $C_D=0.6$  and 1.0) will be provided as supplementary information in the near future. However, it is not anticipated that this information will necessitate additional changes to the Technical Specifications.

The LOCA-ECCS analysis provided in our letter of March 4, 1977, Serial No. 219/082776 remains applicable for steam generator tube plugging up to a level of 19% for Unit No. 1 and 20% for Unit No. 2. The actual steam generator tube plugging level as of this date is less than the above limits of the currently applicable analysis. The reanalysis and associated changes in the Technical Specifications provided in the Attachments along with our previously referenced submittal for the non-LOCA analysis justifies operation with up to 25% of the steam generator tubes being plugged.

All changes have been reviewed by both the Station Nuclear Safety and Operating Committee and the System Nuclear Safety and Operation Committee. Further, it has been determined that these changes do not involve an unreviewed safety question as defined in 10 CFR 50.59.

Should you have any questions or comments, we would be most happy to meet with you at your earliest convenience.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Attachments

cc: Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement

