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Letter to N. C. Moseley from Virginia Electric and Power Company dated July 9, 1975.

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VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

July 9, 1975

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 581
PO&M/JTB:clw

Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Attached is a preliminary report entitled Inservice Inspection Program, Refueling Outage No. 1 for the Surry Power Station, Unit No. 2. The inservice inspection did not identify any significant deficiencies.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Attachment

cc: Mr. K. R. Goller
39 copies

PRELIMINARY RESULTS OF THE

INSERVICE INSPECTION PROGRAM
REFUELING OUTAGE NO. 1
SURRY POWER STATION
UNIT NO. 2

JUNE 15, 1975

REPORT NO. ISI 75-3

DOCKET NO. 50-281
LICENSE NO. DPR-37

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

I. INTRODUCTION

In accordance with the requirements of Technical Specification 6.6.C, this report contains a summary of the preliminary results of the inservice inspection activities performed during the first refueling outage of Unit No. 2 at the Surry Power Station. The document entitled, Inservice Inspection Program, Refueling Outage No. 1, Surry Power Station, Unit No. 2, Report No. ISI 75-2 dated March 10, 1975 provides the specific details concerning the inspections performed.

This report is intended to be a preliminary report of the results of the inservice inspections following the first refueling outage, pursuant to Technical Specification 6.6.C. A more detailed report will be submitted within 90 days.

II. SUMMARY OF INSPECTION RESULTS

The inservice inspections performed during the period covered by this report included all those listed in Report No. ISI 75-2. All inspections were conducted as specified in ISI 75-2 except Technical Specification reference 4.2, regenerative heat exchanger nozzle to shell welds. This was changed from a volumetric examination to a surface examination (liquid penetrant) because the regenerative heat exchanger is a Class III vessel.

All inspections were satisfactory; however, the following indications were identified:

<u>Area Examined</u>	<u>Method of Examination</u>	<u>Indication</u>	<u>Disposition</u>
Gen. A 1.3 (C-301-2501R)	Visual	18 punch marks	Cleared
Dr Vessel 1.8 Core Studs	Visual	Nicks in threads	Cleared
1.2 Miscellaneous Lead SIS Piping	Visual	Arc Strikes	Not significant-will be groundout when time permits
1.3 Miscellaneous Turbine blades	Visual & MT & PT	Arc Strikes, Cracked lashing lugs	Groundout and repaired, all indications were cleared
1.4 Miscellaneous Sized SS	Visual	Arc Strikes at floor supports	Not significant-will be groundout when time permits

III. CONCLUSIONS

The results of the inservice inspections performed verified the integrity of the systems and components examined. The discrepancies noted were corrected or will be corrected as indicated in Part II.

Based on the results of the inservice inspection program, as summarized herein, the safety systems and components inspected have not experienced degradation and there is reasonable assurance that they will continue to perform their design function.

As required by Technical Specification 6.6.C, a detailed report of the inspections performed will be submitted within 90 days.