

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

May 27, 1976



Mr. Norman G. Moseley, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 054  
PO&M/ALH:jlf

Docket No. 50-281  
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. USRE-S2-76-06.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

A handwritten signature in cursive script that reads "G. M. Stallings".

G. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief (40)  
Operating Reactors Branch 4



# LICENSEE EVENT REPORT

CONTROL BLOCK: 

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[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE				EVENT TYPE													
01	V	A	S	P	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	3					
7	8	9	14			15	25						26				30		31	32									
01 CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE															
01	CONT	P	O	L	L	0	5	0	-	0	2	8	1	0	5	0	4	7	6	0	5	2	5	7	6				
7	8	57	58	59	60	61				68				69				74				75				80			

### EVENT DESCRIPTION

02 While Unit Two was at refueling shutdown, two of the three pressurizer level transmit-  
 03 ters were found to have out of specification high level trip setpoints. The high level  
 04 trip setpoints were found as follows: LT-459-93% LT-460-91.56%, & LT461-92.08%; com-  
 05 pared to the Technical Specification value of 92% (T.S.2.3.A.3[a]). Since the trip  
 06 matrix is two out of three, a reactor trip on high pressurizer water level (CONT'D)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION				
07	I	A	E	I	N	S	T	R	U	N	B	O	8	0	Y			
7	8	9	10	11				17		43		44				47		48

### CAUSE DESCRIPTION

08 The cause of this event was normal electronic drift on LT-459 & LT-461 in excess of  
 09 the conservative setting of the setpoint. The initial action was to recalibrate the  
 10 transmitters. The long term corrective action will be to set the trip setpoint (CON'T)

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION								
11	H	0	0	0	N/A	B	N/A									
7	8	9	10		12		13		44		45		46		80	
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE								
12	Z	Z	N/A		N/A											
7	8	9	10		11		44		45		80					

### PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION										
13	0	0	0	Z	N/A									
7	8	9	11		12		13		80					

### PERSONNEL INJURIES

NUMBER		DESCRIPTION										
14	0	0	0	N/A								
7	8	9	11		12		80					

### OFFSITE CONSEQUENCES

15 N/A

### LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION								
16	Z	N/A								
7	8	9	10		80					

### PUBLICITY

17 N/A

### ADDITIONAL FACTORS

18 The health and safety of the general public were not affected by this occurrence  
 19 since a reactor trip would have been initiated .08% above the desired level. (CON'T)

NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

EVENT DESCRIPTION (CONTINUED)

would have been initiated at a level .08% higher than specified. This event is reportable per Technical Specification 6.6.2b(1). (USRE-S2-76-06).

CAUSE DESCRIPTION (CONTINUED)

on pressurizer level on both units an additional 2% conservative to provide further allowance for electronic drift.

ADDITIONAL FACTORS (CONTINUED)

Because of this slight difference in level (less than one half inch), and other methods of protection during an increasing pressurizer level transient (i.e., relief valves and high pressure trip), there would have been no adverse safety implications associated with this event.