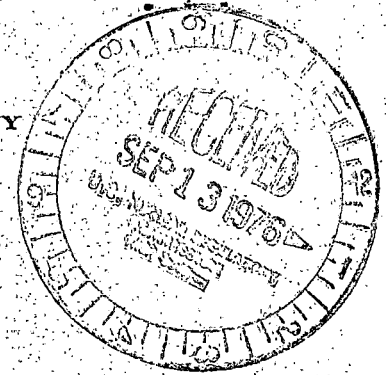


VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 10, 1976



Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 226
PO&M/ALH:clw
Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. USRE-S2-76-12.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief ✓
Operating Reactors Branch 4
(40 copies USRE-S2-76-12)



LICENSEE EVENT REPORT

USRE-S2-76-12

CONTROL BLOCK:

--	--	--	--	--	--

(PLEASE PRINT ALL REQUIRED INFORMATION)

01	V A S P S 2	00-000000-00	41110	03
7 8 9	14	15 25	26 30	31 32

01	CONT	P O	L	L	050-0281	081376	090376
7 8	57 58	59	60	61	68	69 74	75 80

EVENT DESCRIPTION

02 | During normal operation a routine sample of "C" safety injection accumulator indicated
7 8 9

03 | 1893 ppm boron (Time: 1600). This is contrary to the Technical Specification 3.3.A.2
7 8 9

04 | limit of 1950 ppm boron. Accumulator "C" was declared inoperable per Technical Speci-
7 8 9

05 | fication 3.3.B and isolated. After recirculation with the RWST, the boron concentra-
7 8 9

06 | tion increased to 2116 ppm (Time: 1845). A power rampdown was not necessary (con't)
7 8 9

07	P C	E	V A L V E X	A	0020	Y
7 8 9	10	11	12 17	43	44 47	48

CAUSE DESCRIPTION

08 | Leaking of check valves (2-SI-147) and (2-SI-145) caused dilution of the accumulator's
7 8 9

09 | boron concentration from the "C" primary loop. Surveillance of the accumulator's
7 8 9

10 | boron concentration has been increased to weekly and the accumulator is re- (con't)
7 8 9

11	E	100	N/A	B	N/A
7 8 9	9	10 12 13	44	45	46 80

12	Z	Z	N/A	N/A
7 8 9	9	10 11	44	45 80

PERSONNEL EXPOSURES

13	000	Z	N/A
7 8 9	11	12	13 80

PERSONNEL INJURIES

14	000	N/A
7 8 9	11	12 80

OFFSITE CONSEQUENCES

15 | N/A
7 8 9

LOSS OR DAMAGE TO FACILITY

16	Z	N/A
7 8 9	10	80

PUBLICITY

17 | N/A
7 8 9

ADDITIONAL FACTORS

18 | A similar reportable event was experienced with the "B" safety injection accumulator
7 8 9

19 | of Unit No. 1 (USRE-S1-76-08) (July 21, 1976), and analogous correction actions (con't)
7 8 9

NAME: Tyndall L. Baucom PHONE: (804) 357-3184

EVENT DESCRIPTION (con't)

because the accumulator was isolated for a period less than four hours, as permitted by Technical Specification 3.3.B. This event is reportable per Technical Specification 6.6.2.b(1). (USRE-S2-76-12)

CAUSE DESCRIPTION (con't)

circulated with the RWST when necessary.

During the upcoming refueling the check valves will be inspected and repaired as necessary.

The total effect of this boric acid dilution would not have influenced shutdown capability at this point in core life. Any accident analyzed in the safety analysis would not have been affected by the small boron dilution.

ADDITIONAL FACTORS (con't)

were recommended. Since the safety injection system maintained its capabilities, the health and safety of the general public are not affected.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 10, 1976

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

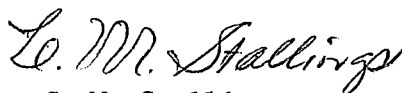
Serial No. 226
PO&M/ALH:clw
Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. USRE-S2-76-12.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,



C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief
Operating Reactors Branch 4
(40 copies USRE-S2-76-12).

LICENSEE EVENT REPORT

USRE-S2-76-12

CONTROL BLOCK:

--	--	--	--	--	--	--	--	--	--

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME				LICENSE NUMBER												LICENSE TYPE				EVENT TYPE					
01	V	A	S	P	S	2	0	0	-	0	0	0	0	0	0	-	0	0	4	1	1	1	0	0	3
7	8	9	14	15	25	26	30	31	32																

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER						EVENT DATE				REPORT DATE											
01	CONT	P	O	L	L	0	5	0	-	0	2	8	1	0	8	1	3	7	6	0	9	0	3	7	6
7	8	57	58	59	60	61	68	69	74	75	80														

EVENT DESCRIPTION

02 | During normal operation a routine sample of "C" safety injection accumulator indicated
7 8 9 |
03 | 1893 ppm boron (Time: 1600). This is contrary to the Technical Specification 3.3.A.2
7 8 9 |
04 | limit of 1950 ppm boron. Accumulator "C" was declared inoperable per Technical Speci-
7 8 9 |
05 | fication 3.3.B and isolated. After recirculation with the RWST, the boron concentra-
7 8 9 |
06 | tion increased to 2116 ppm (Time: 1845). A power rampdown was not necessary (con't)
7 8 9 |

SYSTEM CODE	CAUSE CODE	COMPONENT CODE						PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VIOLATION			
07	P	C	E	V	A	L	V	E	X	A	0	0	2	0	Y
7	8	9	10	11	12	17	43	44	47	48					

CAUSE DESCRIPTION

08 | Leaking of check valves (2-SI-147) and (2-SI-145) caused dilution of the accumulator's
7 8 9 |
09 | boron concentration from the "C" primary loop. Surveillance of the accumulator's
7 8 9 |
10 | boron concentration has been increased to weekly and the accumulator is re- (con't)
7 8 9 |

FACILITY STATUS	% POWER	OTHER STATUS			METHOD OF DISCOVERY	DISCOVERY DESCRIPTION			
11	E	1	0	0	N/A	B	N/A		
7	8	9	10	12	13	44	45	46	80

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY			LOCATION OF RELEASE		
12	Z	Z	N/A	N/A	N/A		
7	8	9	10	11	44	45	80

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION				
13	0	0	0	Z	N/A	
7	8	9	11	12	13	80

PERSONNEL INJURIES

NUMBER	DESCRIPTION				
14	0	0	0	N/A	
7	8	9	11	12	80

OFFSITE CONSEQUENCES

15 | N/A
7 8 9 |

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION			
16	Z	N/A		
7	8	9	10	80

PUBLICITY

17 | N/A
7 8 9 |

ADDITIONAL FACTORS

18 | A similar reportable event was experienced with the "B" safety injection accumulator
7 8 9 |

19 | of Unit No. 1 (USRE-S1-76-08) (July 21, 1976), and analogous correction actions (con't)
7 8 9 |

NAME: Tyndall L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (con't)

because the accumulator was isolated for a period less than four hours, as permitted by Technical Specification 3.3.B. This event is reportable per Technical Specification 6.6.2.b(1). (USRE-S2-76-12)

CAUSE DESCRIPTION (con't)

circulated with the RWST when necessary.

During the upcoming refueling the check valves will be inspected and repaired as necessary.

The total effect of this boric acid dilution would not have influenced shutdown capability at this point in core life. Any accident analyzed in the safety analysis would not have been affected by the small boron dilution.

ADDITIONAL FACTORS (con't)

were recommended. Since the safety injection system maintained its capabilities, the health and safety of the general public are not affected.

263 14 10 31 18

U.S. ATEC
REGULATORY OPERATIONS
REGION II
ATLANTA, GA.

SEP 14 9 18 AM '76