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VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 29, 1976

Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 264
PO&M/ALH:jlf

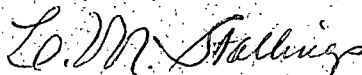
Docket No. 50-281
License No. DPR-37

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Licensee Event Report No. AO-S2-76-05.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

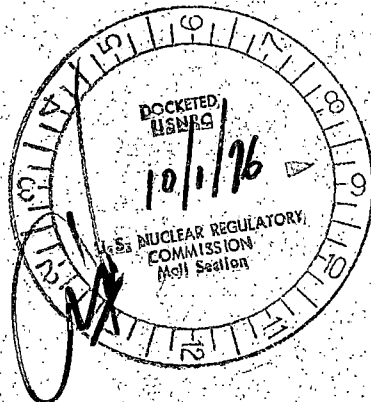
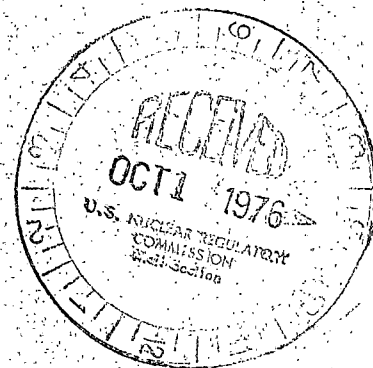
Very truly yours,



C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief (40 copies) ✓
Operating Reactors Branch 4



REGULATORY DOCKET FILE COPY

9942

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE				EVENT TYPE									
01	V	A	S	P	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	1	
7	8	9	14	15	25	26	30	31	32																
CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE			REPORT DATE														
01	CON'T	P	O	T	L	0	5	0	-	0	2	8	1	0	9	1	5	7	6	0	9	2	7	7	6
7	8	57	58	59	60	61	68	69	74	75	80														

EVENT DESCRIPTION

02	During normal operation the pressurizer level fell below the 45% level set point. This																							
7	8	9																						80
03	resulted in a decrease in pressurizer pressure and a decrease in the volume control																							
7	8	9																						80
04	tank level. These events indicated a leak rate in excess of 10 gpm from an unidenti-																							
7	8	9																						80
05	fied source. A reactor trip was manually initiated per Emergency Procedure #4 (Time:																							
7	8	9																						80
06	1345). A manual safety injection followed the manual trip per Emergency Procedure (Cont																							
7	8	9																						80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER			VIOLATION		
07	H	J	F	X	X	X	X	X	X	N	W	1	2	0	Y
7	8	9	10	11	12	17	43	44	47	48					

CAUSE DESCRIPTION

08	Steam Generator "A" of Unit 2 was isolated via Phase I isolation and a leak rate in																							
7	8	9																						80
09	excess of 10gpm was confirmed (9/15/76) (Time: 1400). Operations personnel proceeded																							
7	8	9																						80
10	to bring the reactor to a cold shutdown condition. The tube leak was visually (Con't)																							
7	8	9																						80

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
11	G	1	0	0	N/A	A	Operational Event						
7	8	9	10	12	13	44	45	46					80

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE					
12	Z	Z	N/A				N/A						
7	8	9	10	11	44	45					80		

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION		
13	0	0	0	Z	N/A	
7	8	9	11	12	13	80

PERSONNEL INJURIES

NUMBER		DESCRIPTION			
14	0	0	0	N/A	
7	8	9	11	12	80

OFFSITE CONSEQUENCES

15	N/A																							
7	8	9																						80

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION		
16	L	N/A		
7	8	9	10	80

PUBLICITY

17	N/A																							
7	8	9																						80

ADDITIONAL FACTORS

18	Since the reactor was brought to a cold shutdown condition after the detection of the																							
7	8	9																						80
19	leak and the major portion of the gaseous activity from the air ejector was (continued)																							
7	8	9																						80

NAME: T. L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (con't)

#4. This event is reportable per Technical Specification 3.1.C.4. (AO-S2-76-05).

CAUSE DESCRIPTION (con't)

confirmed in steam generator "A", Row 1 and Column 7, (Sept. 17, 1976, Time: 1711). Corrective measures have been started to evaluate the cause of the tube failure and to plug the leaking tube in steam generator "A".

ADDITIONAL FACTORS (con't)

vented to containment, the health and safety of the general public were not affected.

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 29, 1976

Mr. Norman G. Moseley, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
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License No. DPR-37

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The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

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Vice President-Power Supply
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief (40 copies)
Operating Reactors Branch 4

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME:

01	V	A	S	P	S	2
----	---	---	---	---	---	---

 LICENSE NUMBER:

0	0	-	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---

 LICENSE TYPE:

4	1	1	1	0
---	---	---	---	---

 EVENT TYPE:

0	1
---	---

CONT:

P	O
---	---

 REPORT TYPE:

T

 REPORT SOURCE:

L

 DOCKET NUMBER:

0	5	0	-	0	2	8	1
---	---	---	---	---	---	---	---

 EVENT DATE:

0	9	1	5	7	6
---	---	---	---	---	---

 REPORT DATE:

0	9	2	7	7	6
---	---	---	---	---	---

EVENT DESCRIPTION

02 | During normal operation the pressurizer level fell below the 45% level set point. This
03 | resulted in a decrease in pressurizer pressure and a decrease in the volume control
04 | tank level. These events indicated a leak rate in excess of 10 gpm from an unidenti-
05 | fied source. A reactor trip was manually initiated per Emergency Procedure #4 (Time:
06 | 1345). A manual safety injection followed the manual trip per Emergency Procedure (Cont)

SYSTEM CODE:

H	J
---	---

 CAUSE CODE:

F

 COMPONENT CODE:

X	X	X	X	X	X
---	---	---	---	---	---

 PRIME COMPONENT SUPPLIER:

N

 COMPONENT MANUFACTURER:

W	1	2	0
---	---	---	---

 VIOLATION:

Y

CAUSE DESCRIPTION

08 | Steam Generator "A" of Unit 2 was isolated via Phase I isolation and a leak rate in
09 | excess of 10gpm was confirmed (9/15/76) (Time: 1400). Operations personnel proceeded
10 | to bring the reactor to a cold shutdown condition. The tube leak was visually (Con't)

FACILITY STATUS:

G

 % POWER:

1	0	0
---	---	---

 OTHER STATUS:

N/A

 METHOD OF DISCOVERY:

A

 DISCOVERY DESCRIPTION:

Operational Event

FORM OF ACTIVITY RELEASED:

Z

 CONTENT OF RELEASE:

Z

 AMOUNT OF ACTIVITY:

N/A

 LOCATION OF RELEASE:

N/A

PERSONNEL EXPOSURES

NUMBER:

0	0	0
---	---	---

 TYPE:

Z

 DESCRIPTION:

N/A

PERSONNEL INJURIES

NUMBER:

0	0	0
---	---	---

 DESCRIPTION:

N/A

OFFSITE CONSEQUENCES

15 | N/A

LOSS OR DAMAGE TO FACILITY

TYPE:

L

 DESCRIPTION:

N/A

PUBLICITY

17 | N/A

ADDITIONAL FACTORS

18 | Since the reactor was brought to a cold shutdown condition after the detection of the
19 | leak and the major portion of the gaseous activity from the air ejector was (continued)

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PHONE: (804) 357-3184

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U.S.A.E.C.
REGULATORY OPERATIONS
REGION II
ATLANTA, GA.

Oct. 1 9 32 AM '76