

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

August 11, 1977



Regulatory

ELG:CS

Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 338  
PO&M/TAP:das  
Docket No. 50-281  
License No. DPR-37

Dear Mr. O'Reilly:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. RO-S2-77-05.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosures

40 copies RO-S2-77-05

cc: Mr. Robert W. Reid, Chief  
Operating Reactors Branch 4

# LICENSEE EVENT REPORT

CONTROL BLOCK: 

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(PLEASE PRINT ALL REQUIRED INFORMATION)

01	V	A	S	P	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	0	0	3		
7	8	9	14	15	25	28	30	31	32																
LICENSEE NAME												LICENSE NUMBER						LICENSE TYPE				EVENT TYPE			
01	CONT	P	O	L	L	0	5	0	-	0	2	8	1	0	7	1	8	7	7	0	8	0	3	7	7
7	8	57	58	59	60	61	68	69	74	75	80														
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER							EVENT DATE				REPORT DATE								

**EVENT DESCRIPTION**

02 | At Cold Shutdown, in the course of routine testing, Reactor Coolant Flow Comparator  
 03 | (FC-2-414) failed to trip. The two redundant systems were operational for the  
 04 | required two out of three low flow trip. Previous test was satisfactory. The  
 05 | corrective action implemented was to replace the comparator with an operational  
 06 | spare. This condition is reportable per Technical Specification 6.6.2b(2).

07	I	A	E	I	N	S	T	R	U	N	H	0	2	0	N	
7	8	9	10	11	12	17	43	44	47	48						
SYSTEM CODE		CAUSE CODE		COMPONENT CODE						PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER			VIOLATION	

(RO-S2-77-5 )

**CAUSE DESCRIPTION:**

08 | A Westinghouse Hagan Single Signal Comparator Model 4111082-001 failed to trip due to  
 09 | capacitor leakage. Since these capacitors have an expected life of 10 to 20 years,  
 10 | and this failed capacitor has been in use for approximately nine years, the (Cont'd)

11	G	0	0	0	N/A	B	PT-2.2	-	Reactor Coolant Flow															
7	8	9	10	12	13	44	45	46	80															
FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY																	
DISCOVERY DESCRIPTION		FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE																
12	Z	Z	N/A	N/A	N/A	N/A	N/A	N/A	N/A															
7	8	9	10	11	44	45																		

**PERSONNEL EXPOSURES**

13	0	0	0	Z	N/A
7	8	9	11	12	13
NUMBER		TYPE		DESCRIPTION	

**PERSONNEL INJURIES**

14	0	0	0	N/A
7	8	9	11	12
NUMBER		DESCRIPTION		

**OFFSITE CONSEQUENCES**

15 | N/A

**LOSS OR DAMAGE TO FACILITY**

16	Z	N/A
7	8	9
TYPE		DESCRIPTION

**PUBLICITY**

17 | N/A

**ADDITIONAL FACTORS**

18 | The health and safety of the public were not affected because the two operational  
 19 | redundant comparators would have initiated the required low flow trip, in that (Cont'd.)

NAME: J. L. Rouman PHONE: 804-357-3184

CAUSE DESCRIPTION (CONTINUED)

probable cause was aging. As per a Westinghouse Technical Bulletin (NSD-TB-75-14) dated October 21, 1975, these capacitors have no specific problem of a generic nature. However, Westinghouse does recommend that these leaking capacitors be replaced with a premium grade or Tantalum capacitor, presently available, on an as-failed basis, instead of a general capacitor replacement program. This criteria is presently observed at Surry. The present rate of capacitor failure is very low and any increase in this rate will require a re-evaluation of our capacitor change-out policy.

ADDITIONAL FACTORS (CONTINUED)

particular loop.

RECEIVED DOCUMENT  
PROCESSING UNIT

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