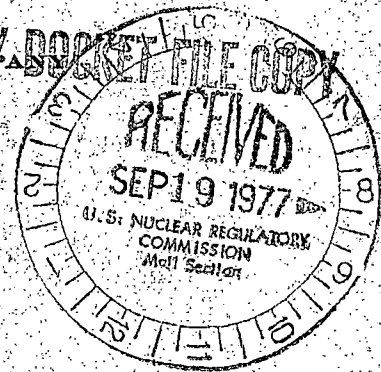


VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

September 15, 1977



Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 411
POSM/TAP:dgt
Docket No. 50-281

License No. DPR-37

Dear Mr. O'Reilly:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. RO-S2-77-07..

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

C. M. Stallings

C. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosures

40 copies RO-S2-77-07

cc: Mr. Robert W. Reid, Chief
Operating Reactors Branch 4

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

7	8	9	14	15	25	26	30	31	32															
01	V	A	S	P	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	3
LICENSEE NAME			LICENSE NUMBER						LICENSE TYPE				EVENT TYPE											

7	8	57	58	59	60	61	68	69	74	75	80														
01	CONT	M	I	L	L	0	5	0	-	0	2	8	1	0	8	1	5	7	7	0	9	1	4	7	7
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER						EVENT DATE			REPORT DATE										

EVENT DESCRIPTION

02 | During normal operation, routine sampling disclosed that Boron Injection Tank and "C" Boric Acid Storage Tank concentrations were low: BIT 10.9% and "C" BAST 11.0%. This was a violation of Technical Specifications 3.2.C.3 and 3.3.A.3. Unit rampdown was commenced immediately and BIT recirculation shifted to "B" BAST which was 11.9% concentration. Recirculation returned the BIT to 11.6% and the unit was returned to full

(Continued)

7	8	9	10	11	12	17	43	44	47	48					
07	P	C	E	V	A	L	V	E	X	N	G	2	5	5	Y
SYSTEM CODE		CAUSE CODE		COMPONENT CODE			PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VIOLATION				

CAUSE DESCRIPTION

08 | Dilution was caused by a cracked diaphragm on valve 1-CH-144; PG supply to "D" BATP suction. The valve was isolated and repaired. Grinnell diaphragm valves at this station are now on a routine replacement program based on time in service. This

(Continued)

7	8	9	10	12	13	44	45	46	80
11	E	1	0	0	N/A	B	N/A		
FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION

7	8	9	10	11	44	45	80
12	Z	Z	N/A				
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	

PERSONNEL EXPOSURES

7	8	9	11	12	13	80
13	0	0	0	Z	N/A	
NUMBER		TYPE		DESCRIPTION		

PERSONNEL INJURIES

7	8	9	11	12	80
14	0	0	0	N/A	
NUMBER		DESCRIPTION			

OFFSITE CONSEQUENCES

15 | N/A

LOSS OR DAMAGE TO FACILITY

7	8	9	10	80
16	Z	N/A		
TYPE		DESCRIPTION		

PUBLICITY

17 | N/A

ADDITIONAL FACTORS

18 | The health and safety of the general public was not affected.

18 |

772620114

NAME: T. L. T. [Signature]

PHONE: (804) 357-3184

EVENT DESCRIPTION (CONTINUED)

power. Subsequently, "C" BAST concentration was enriched to 12.3%. This is reportable per Technical Specification 6.6.2.b(2). (RO-S2-77-07).

CAUSE DESCRIPTION (CONTINUED)

program should prevent further failures of this type.

RECEIVED DOCUMENT
PROCESSING UNIT

1977 SEP 19 AM 9 30