

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

October 11, 1977



Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

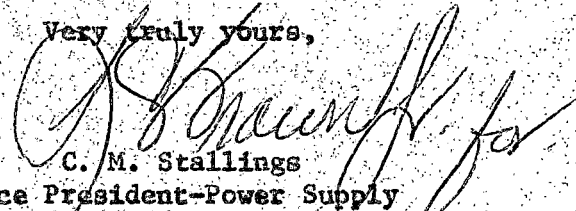
Serial No. 449  
PO&M/TAP:dgt  
Docket No. 50-281  
License No. DFR-37

Dear Mr. O'Reilly:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. RO-S2-77-09.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

  
C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosures

40 copies RO-S2-77-09

cc: Mr. Robert W. Reid, Chief  
Operating Reactors Branch 4

772870141

# LICENSEE EVENT REPORT

CONTROL BLOCK: 

--	--	--	--	--	--

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME: 

01	V	A	S	P	S	2
----	---	---	---	---	---	---

 LICENSE NUMBER: 

0	0	-	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---

 LICENSE TYPE: 

4	1	1	1	0
---	---	---	---	---

 EVENT TYPE: 

0	3
---	---

CATEGORY: 

01	CONT
----	------

 REPORT TYPE: 

L
---

 REPORT SOURCE: 

L
---

 DOCKET NUMBER: 

0	5	0	-	0	2	8	1
---	---	---	---	---	---	---	---

 EVENT DATE: 

0	9	1	0	7	7
---	---	---	---	---	---

 REPORT DATE: 

1	0	0	6	7	7
---	---	---	---	---	---

### EVENT DESCRIPTION

02
----

 During a refueling shutdown P.T.-13, test of Main Steam Safety Valve Setpoint, indicated five (5) valves were not within plus or minus three (3%) percent of the setpoint as required by the periodic test. The corrective action taken was to reset the valves (SV-MS-201A, 201B, 201C, 202B, 202C) this event is reportable per Technical Specification 6.6.2b (RO-S2-77-09)

SYSTEM CODE: 

C	C
---	---

 CAUSE CODE: 

F
---

 COMPONENT CODE: 

V	A	L	V	E	X
---	---	---	---	---	---

 PRIME COMPONENT SUPPLIER: 

A
---

 COMPONENT MANUFACTURER: 

C	5	6	7
---	---	---	---

 VIOLATION: 

Y
---

### CAUSE DESCRIPTION

08
----

 Five (5) Consolidated safety relief valves were found to be outside of the 3% as found tolerance in their pop point. The cause is attributable to normal setpoint drift. The valves were adjusted and retested satisfactorily.

FACILITY STATUS: 

G
---

 % POWER: 

0	0	0
---	---	---

 OTHER STATUS: 

NA
----

 METHOD OF DISCOVERY: 

B
---

 DISCOVERY DESCRIPTION: 

NA
----

FORM OF ACTIVITY RELEASED: 

Z
---

 CONTENT OF RELEASE: 

Z
---

 AMOUNT OF ACTIVITY: 

NA
----

 LOCATION OF RELEASE: 

NA
----

### PERSONNEL EXPOSURES

NUMBER: 

0	0	0
---	---	---

 TYPE: 

Z
---

 DESCRIPTION: 

NA
----

### PERSONNEL INJURIES

NUMBER: 

0	0	0
---	---	---

 DESCRIPTION: 

NA
----

### OFFSITE CONSEQUENCES

15
----

 NA

### LOSS OR DAMAGE TO FACILITY

TYPE: 

Z
---

 DESCRIPTION: 

NA
----

### PUBLICITY

17
----

 NA

### ADDITIONAL FACTORS

18
----

 The combined capacity of the safety valves is designed to exceed the total steam flow corresponding to the maximum steady state power that can be obtained during one, 

19
----

NAME: T. L. Baucom PHONE: (804) 357-3184

ADDITIONAL FACTORS (CONTINUED)

two, or three loop operation. Full relief capacity would have been achieved at 1145 PSIG, 1133 PSIG, and 1140 PSIG for loops A, B, and C respectively, contrary to the design value of 1135 PSIG. This capability would have prevented any overpressurization of the steam generators. Hence, this drift of setpoint did not endanger the health and safety of the general public.

RECEIVED DOCUMENT  
PROCESSING UNIT

1977 OCT 14 PM 12 26