

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

September 8, 1975

REGULATORY DOCKET FILE COPY

Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 647-S  
PO&M/JTB:clw

Docket No. 50-280  
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.B.1, the Virginia Electric and Power Company hereby submits forty (40) copies of supplementary information to Abnormal Occurrence Report No. AO-S1-75-15. This information was requested by the NRC.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

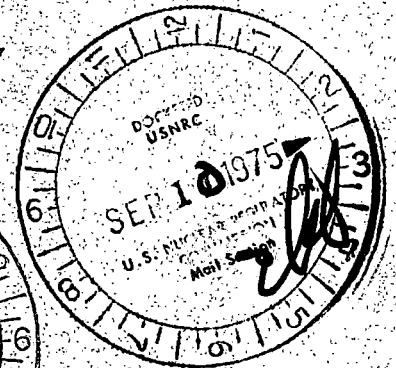
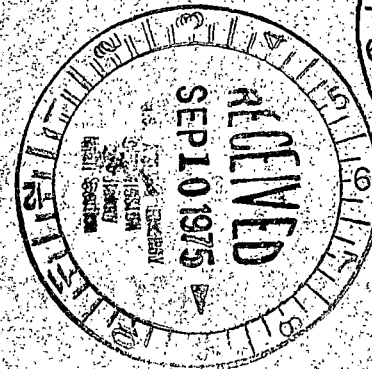
*C. M. Stallings*

C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosures  
40 copies of AO-S1-75-15

cc: Mr. K. R. Goller ✓

Mr. Bryce P. Schofield  
Director, Bureau of Industrial  
Hygiene  
Department of Health



9440

# LICENSEE EVENT REPORT

Updated Report of

A0-S1-75-15

Previous Report Dated 8-15-75

(PLEASE PRINT ALL REQUIRED INFORMATION)

CONTROL BLOCK: 

--	--	--	--	--	--

LICENSEE NAME: 

01	V	A	S	P	S	1
----	---	---	---	---	---	---

 LICENSE NUMBER: 

0	0	-	0	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---	---

 LICENSE TYPE: 

4	1	1	1	0
---	---	---	---	---

 EVENT TYPE: 

0	1
---	---

CATEGORY: 

01	CONT
----	------

 REPORT TYPE: 

P	O
---	---

 REPORT SOURCE: 

T	L
---	---

 DOCKET NUMBER: 

0	5	0	-	0	2	8	0
---	---	---	---	---	---	---	---

 EVENT DATE: 

0	8	0	3	7	5
---	---	---	---	---	---

 REPORT DATE: 

0	9	0	3	7	5
---	---	---	---	---	---

### EVENT DESCRIPTION

02 | During normal operation a leaking fitting on "B" overhead gas compressor resulted in  
03 | the unplanned release of radioactive material from the site. Appropriate operational  
04 | procedures were followed to isolate the release. The leaking fitting has been re-  
05 | paired. (A0-S1-75-15)  
06 |

SYSTEM CODE: 

M	B
---	---

 CAUSE CODE: 

E
---

 COMPONENT CODE: 

P	I	P	E	X	X
---	---	---	---	---	---

 PRIME COMPONENT SUPPLIER: 

A
---

 COMPONENT MANUFACTURER: 

Z	9	9	9
---	---	---	---

 VIOLATION: 

Y
---

### CAUSE DESCRIPTION

08 | A loose fitting on "B" overhead gas compressor allowed airborne radioactive material  
09 | to leak into the gas stripper room and thus offsite via the ventilation vent system.  
10 |

FACILITY STATUS: 

E
---

 % POWER: 

1	0	0
---	---	---

 OTHER STATUS: 

N/A
-----

 METHOD OF DISCOVERY: 

A
---

 DISCOVERY DESCRIPTION: 

Ventilation Vent Monitor Alarm
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FORM OF ACTIVITY RELEASED: 

M
---

 CONTENT OF RELEASE: 

M
---

 AMOUNT OF ACTIVITY: 

28.5 Ci Total
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 LOCATION OF RELEASE: 

Auxiliary Bldg. Vent to Atmosphere
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### PERSONNEL EXPOSURES

13 | NUMBER: 

0	0	0
---	---	---

 TYPE: 

Z
---

 DESCRIPTION: 

N/A
-----

### PERSONNEL INJURIES

14 | NUMBER: 

0	0	0
---	---	---

 DESCRIPTION: 

N/A
-----

### OFFSITE CONSEQUENCES

15 | N/A

### LOSS OR DAMAGE TO FACILITY

16 | TYPE: 

Z
---

 DESCRIPTION: 

N/A
-----

### PUBLICITY

17 | N/A

### ADDITIONAL FACTORS

18 | See attached.

19 |

NAME: E. M. Sweeney, Jr.

PHONE: (804) 357-3184

UPDATED REPORT  
RELEASE OF AIRBORNE RADIOACTIVE  
MATERIAL DUE TO OVERHEAD GAS COMPRESSOR LEAK

ABNORMAL OCCURRENCE AO-S1-75-15

This information is provided in response to a request by the NRC on August 21, 1975.

Health physics personnel were notified that an unplanned release of airborne radioactive material was in progress at 1815 on August 3, 1975. Personnel began taking qualitative portable air samples as soon as the radiation monitor alarm was annunciated. Quantitative air sampling results were initiated at approximately 0800 on August 4, 1975, with the decontamination building being the suspect area. Results were negative. The gas stripper room was sampled at 1100 hours with results being positive. It was then found that the ventilation vent sampler location for the radiation monitor indicating decontamination building was erroneous.

Tritium samples were not taken. Past experience had revealed that  $I^{131}$  is the limiting nuclide and; therefore, no tritium sampling was conducted.

It is not likely that limits of Appendix B, Table I to 10 CFR 20 were exceeded in the auxiliary building. The highest actual measured concentrations of radionuclides listed in the basic report support this conclusion.