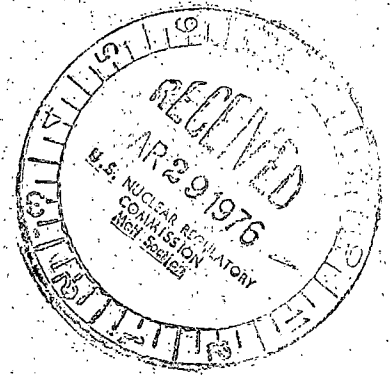
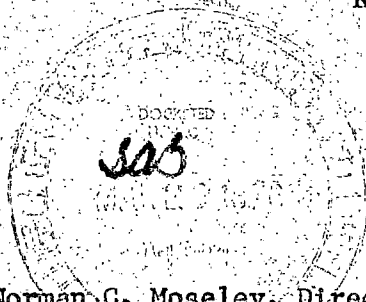


VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

March 26, 1976



Mr. Norman C. Moseley, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region II - Suite 818
230 Peachtree Street, Northwest
Atlanta, Georgia 30303

Serial No. 950
PO&M/ALH:jlf

Docket No. 50-280
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits forty (40) copies of Reportable Occurrence No. AO-S1-76-02.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

G. M. Stallings
Vice President-Power Supply
and Production Operations

Enclosures

40 copies AO-S1-76-02

cc: Mr. Robert W. Reid, Chief
Operating Reactors Branch 4

3119

LICENSEE EVENT REPORT

A0-S1-76-02

CONTROL BLOCK:

1	2	3	4	5	6
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[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE				EVENT TYPE									
01	V	A	S	P	S	1	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	1	
7	8	9	14	15	25	26	30	31	32																
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER				EVENT DATE				REPORT DATE											
01	CONT	M	I	T	L	0	5	0	-	0	2	8	0	0	3	1	3	7	6	0	3	2	6	7	6
7	8	57	58	59	60	61	68	69	74	75	80														

EVENT DESCRIPTION

02	The worst break LOCA core thermal transient analysis is inaccurate in that it does																							80
03	not include the effects of steam generator tube plugging. This is reported pur-																							80
04	suant to Technical Specification 6.6.2.a.8 (A0-S1-76-02)																							80
05																								80
06																								80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION				
07	Z	Z	F	H	T	E	X	C	H	N	W	1	2	0	N			
7	8	9	10	11	12	17	43	44	47	48								

CAUSE DESCRIPTION

08	Steam generator tube deterioration has resulted in the need to plug a number of tubes																							80
09	in Surry steam generators. Surry Unit No. 1 has 5.19% of its steam generator tubes																							80
10	plugged, and Surry Unit No. 2 has 3.37% of its tubes plugged. The cause for (cont'd)																							80

FACILITY STATUS		% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
11	Z	0	0	0	NA	B	NA							
7	8	9	10	12	13	44	45	46						

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE				
12	Z	Z	NA					NA				
7	8	9	10	11	44	45						

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION	
13	0	0	Z	NA	
7	8	9	11	12	13

PERSONNEL INJURIES

NUMBER		DESCRIPTION		
14	0	0	NA	
7	8	9	11	12

OFFSITE CONSEQUENCES

15	There are no offsite consequences associated with this event.																							80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION	
16	Z	NA	
7	8	9	10

PUBLICITY

17	NA																							80
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ADDITIONAL FACTORS

18	The nuclear steam supply system vendor has recently performed a LOCA analysis to																							80
19	examine the effects of plugging 6% of the steam generator tubes on each Surry unit.																							80

NAME: E. M. Sweeney, Jr. PHONE: (804) 357-3184

CAUSE DESCRIPTION (CONT'D)

steam generator tube deterioration at Surry has not been isolated. The nuclear steam supply system vendor is conducting exhaustive research into this generic problem with PWR steam generators.

ADDITIONAL FACTORS (CONT'D)

The major effect of the plugged tubes occurs during the reflooding phase whereby increasing loop flow resistance results in decreasing flooding rate.

The approved ECCS Evaluation Model (March 15, 1975 version) was used to perform the analysis using the sequence SATAN/WREFLOOD/LOCTA. The resulting peak clad temperature for 6% plugged tubes was 2142°F using the current technical specification F_0 limit of 2.1. These results show that the ECCS criteria, peak clad temperature 2200°F, are met for Surry Unit Nos. 1 and 2 with the inclusion of 6% plugged tubes.

Therefore, the health and safety of the general public was not affected by this occurrence.

Follow up analysis is being conducted by the nuclear steam supply system vendor. This analysis will define the allowable percentage of steam generator tubes that may be plugged in each Surry unit. A supplementary report will be submitted when the results of this analysis are known.