

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

September 3, 1976

Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 214  
PO&M/ALH:clw

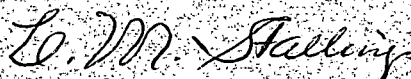
Docket No. 50-280  
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Licensee Event Report No. AO-S1-76-05.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

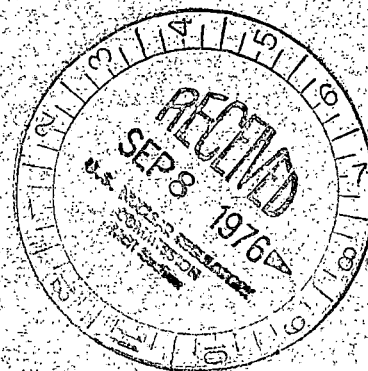
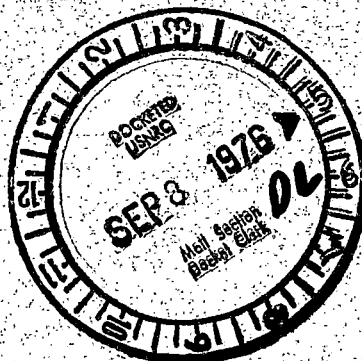
Very truly yours,



C. M. Stallings  
Vice President - Power Supply  
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief ✓  
Operating Reactors Branch 4  
(40 copies AO-S1-76-05)



LICENSEE EVENT REPORT

A0-S1-76-05

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

Regulatory Docket File

|                                 |   |   |   |    |    |    |    |    |                           |  |  |                   |  |
|---------------------------------|---|---|---|----|----|----|----|----|---------------------------|--|--|-------------------|--|
| LICENSEE NAME<br>01 V A S P S 1 |   |   | LICENSE NUMBER<br>0 0 - 0 0 0 0 0 - 0 0 |    |    |    |    |    | LICENSE TYPE<br>4 1 1 1 0 |  |  | EVENT TYPE<br>0 1 |  |
| 7                               | 8 | 9 | 14                                      | 15 | 25 | 26 | 30 | 31 | 32                        |  |  |                   |  |

|                      |   |                    |    |                      |    |                                  |    |    |    |                           |    |  |                            |  |  |
|----------------------|---|--------------------|----|----------------------|----|----------------------------------|----|----|----|---------------------------|----|--|----------------------------|--|--|
| CATEGORY<br>01 CON'T |   | REPORT TYPE<br>D I |    | REPORT SOURCE<br>T L |    | DOCKET NUMBER<br>0 5 0 - 0 2 8 0 |    |    |    | EVENT DATE<br>0 8 0 5 7 6 |    |  | REPORT DATE<br>0 8 2 0 7 6 |  |  |
| 7                    | 8 | 57                 | 58 | 59                   | 60 | 61                               | 68 | 69 | 74 | 75                        | 80 |  |                            |  |  |

EVENT DESCRIPTION

02 The Virginia Electric and Power Company was informed by its nuclear steam supply system vendor that an error was discovered in the loss of coolant accident ECCS analysis. It was determined that the reactor vessel upper head water temperature was assigned too low a value. More detailed investigation showed that the assignment of the NRC suggested temperature value would require a reduction in the F<sub>Q</sub> limit. (con't)

|                    |   |                 |    |                               |    |    |    |                               |                                   |    |  |                |  |
|--------------------|---|-----------------|----|-------------------------------|----|----|----|-------------------------------|-----------------------------------|----|--|----------------|--|
| SYSTEM CODE<br>Z Z |   | CAUSE CODE<br>F |    | COMPONENT CODE<br>Z Z Z Z Z Z |    |    |    | PRIME COMPONENT SUPPLIER<br>N | COMPONENT MANUFACTURER<br>W 1 2 0 |    |  | VIOLATION<br>N |  |
| 7                  | 8 | 9               | 10 | 11                            | 12 | 17 | 43 | 44                            | 47                                | 48 |  |                |  |

CAUSE DESCRIPTION

08 The reactor upper internals were designed to permit a small amount of reactor coolant cold leg flow to bypass the core and cool the water in the upper head area. It was, therefore, assumed that the upper head water temperature would be equal to (con't)

|                                |   |                         |    |                           |    |    |                          |                            |   |  |  |  |    |
|--------------------------------|---|-------------------------|----|---------------------------|----|----|--------------------------|----------------------------|---|--|--|--|----|
| FACILITY STATUS<br>E           |   | % POWER<br>1 0 0        |    | OTHER STATUS<br>N/A       |    |    | METHOD OF DISCOVERY<br>D |                            | DISCOVERY DESCRIPTION<br>Notification by Vendor |  |  |  |    |
| 7                              | 8 | 9                       | 10 | 12                        | 13 | 44 | 45                       | 46                         |   |  |  |  | 80 |
| FORM OF ACTIVITY RELEASED<br>Z |   | CONTENT OF RELEASE<br>Z |    | AMOUNT OF ACTIVITY<br>N/A |    |    |                          | LOCATION OF RELEASE<br>N/A |   |  |  |  |    |
| 7                              | 8 | 9                       | 10 | 11                        | 44 | 45 |                          |                            |   |  |  |  | 80 |

PERSONNEL EXPOSURES

|                 |   |           |                    |    |    |
|-----------------|---|-----------|--------------------|----|----|
| NUMBER<br>0 0 0 |   | TYPE<br>Z | DESCRIPTION<br>N/A |    |    |
| 7               | 8 | 9         | 11                 | 12 | 13 |

PERSONNEL INJURIES

|                 |   |                    |    |    |
|-----------------|---|--------------------|----|----|
| NUMBER<br>0 0 0 |   | DESCRIPTION<br>N/A |    |    |
| 7               | 8 | 9                  | 11 | 12 |

OFFSITE CONSEQUENCES

15 No offsite consequences resulted from this event.

LOSS OR DAMAGE TO FACILITY

|           |                    |   |    |
|-----------|--------------------|---|----|
| TYPE<br>Z | DESCRIPTION<br>N/A |   |    |
| 7         | 8                  | 9 | 10 |

PUBLICITY

17 N/A

ADDITIONAL FACTORS

18 This event is the subject of a detailed technical report forwarded to the NRC under

19 Virginia Electric and Power Company letter serial number 194, dated August 18, (con't)

NAME: Tyndall L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (con't)

to a value specified by the NRC Division of Licensing. This is necessary to prevent peak clad temperature from exceeding 2200 degrees F during the design basis accident. This event is reportable per Technical Specification 6.6.2.a.8 (AO-S1-76-05).

CAUSE DESCRIPTION (con't)

reactor cold leg temperature. Recent model results and a spot check on one operating plant showed that the upper head water temperature more closely approaches reactor hot leg temperature.

ADDITIONAL FACTORS (con't)

1976, and serial number 211 dated August 26, 1976. This report presents the operational restrictions that have been applied to Surry Unit Nos. 1 and 2 as a result of this event. The health and safety of the public was not affected by this event.