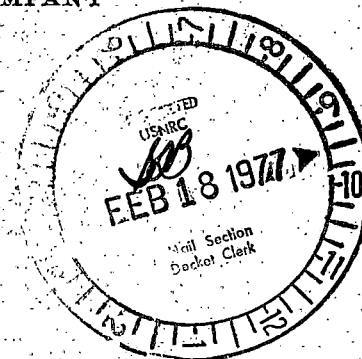


VIRGINIA ELECTRIC AND POWER COMPANY  
 RICHMOND, VIRGINIA 23261

February 16, 1977



Mr. Norman G. Moseley, Director  
 Office of Inspection and Enforcement  
 U. S. Nuclear Regulatory Commission  
 Region II - Suite 818  
 230 Peachtree Street, Northwest  
 Atlanta, Georgia 30303

Serial No. 062  
 P&M/TAP:dgt  
 Docket No. 57-280  
 License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Reportable Occurrence No. RO-S1-77-04.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
 Vice President-Power Supply  
 and Production Operations

Enclosure

40 copies RO-S1-77-04

cc: Mr. Robert W. Reid, Chief ✓  
 Operating Reactors Branch 4



# LICENSEE EVENT REPORT

CONTROL BLOCK: 

--	--	--	--	--	--

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 

01	V	A	S	P	S	I
----	---	---	---	---	---	---

 LICENSE NUMBER: 

0	0	-	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---

 LICENSE TYPE: 

4	1	1	1	0
---	---	---	---	---

 EVENT TYPE: 

0	3
---	---

CATEGORY: 

01	CONT
----	------

 REPORT TYPE: 

L
---

 REPORT SOURCE: 

L
---

 DOCKET NUMBER: 

0	5	0	-	0	2	8	0
---	---	---	---	---	---	---	---

 EVENT DATE: 

0	1	2	6	7	7
---	---	---	---	---	---

 REPORT DATE: 

--	--	--	--	--	--	--	--	--	--

## EVENT DESCRIPTION

02 | During power escalation for Surry 1, cycle 4 refueling startup, the reactor exceeded | 80  
03 | the 90% power hold point prior to the recalibration of the excore symmetrical off-set | 80  
04 | detector system to in-core instrumentation systems as required by the Technical | 80  
05 | Specification 3.18B. This was an administrative error since personnel went to the p. | 80  
06 | threshold hold point at 92% power and omitted the 90% Nuclear Instrumentation(NI)(Cont'd) | 80

SYSTEM CODE: 

Z	Z
---	---

 CAUSE CODE: 

A
---

 COMPONENT CODE: 

Z	Z	Z	Z	Z	Z
---	---	---	---	---	---

 PRIME COMPONENT SUPPLIER: 

Z
---

 COMPONENT MANUFACTURER: 

Z	9	9	9
---	---	---	---

 VIOLATION: 

y
---

## CAUSE DESCRIPTION

08 | The unit was holding power below 90% until the NI Calibration was completed and the | 80  
09 | value of P-Threshold had been determined. Upon notification of the 92% value for | 80  
10 | P-threshold, the unit was erroneously ramped to that power level. Upon (Cont'd) | 80

FACILITY STATUS: 

B
---

 % POWER: 

0	9	2
---	---	---

 OTHER STATUS: 

NA
----

 METHOD OF DISCOVERY: 

A
---

 DISCOVERY DESCRIPTION: 

NA
----

FORM OF ACTIVITY RELEASED: 

Z
---

 CONTENT OF RELEASE: 

Z
---

 AMOUNT OF ACTIVITY: 

NA
----

 LOCATION OF RELEASE: 

--

## PERSONNEL EXPOSURES

NUMBER: 

0	0	0
---	---	---

 TYPE: 

Z
---

 DESCRIPTION: 

NA
----

## PERSONNEL INJURIES

NUMBER: 

0	0	0
---	---	---

 DESCRIPTION: 

NA
----

## OFFSITE CONSEQUENCES

15 | NA | 80

## LOSS OR DAMAGE TO FACILITY

TYPE: 

Z
---

 DESCRIPTION: 

NA
----

## PUBLICITY

17 | NA | 80

## ADDITIONAL FACTORS

18 | The health and safety of the general public were not affected by this occurrence | 80

19 | | 80

NAME: T. L. Baucom

PHONE: (804) 357-3184

EVENT DESCRIPTION (CONTINUED)

calibration hold point. The plant was at 92% power for approximately one hour before the error was noticed and an immediate reduction of power to 90% was implemented. This condition is reportable per Technical Specification 6.6.2b(3) (RO-S1-77-04).

CAUSE DESCRIPTION (CONTINUED)

realization that NI Calibration was not completed, the unit's power level was reduced to 90%. All personnel involved with the occurrence have been appropriately counseled relative to this occurrence, which should preclude recurrence.