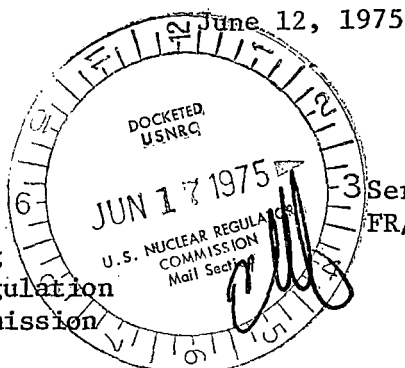


Vepco

REGULATORY DIVISION TELETYPE COPY

VIRGINIA ELECTRIC AND POWER COMPANY, RICHMOND, VIRGINIA 23261



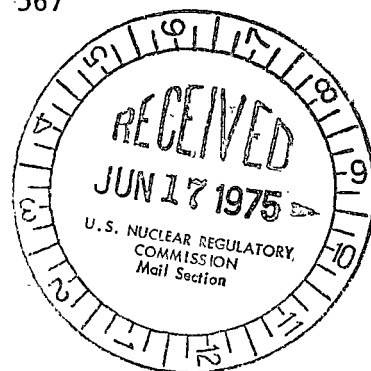
50-280

50-281

Mr. Robert A. Purple, Chief
Operating Reactors Branch #1
Division of Reactor Licensing
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Purple:

SUPPLEMENTAL INFORMATION TO PROPOSED
TECHNICAL SPECIFICATIONS CHANGE NO. 29
SUBMITTED PURSUANT TO 10 CFR 50.46,
SURRY POWER STATION, UNITS NO. 1 & 2



On June 6, 1975, pursuant to 10 CFR 50.46, we submitted an ECCS evaluation along with associated proposed technical specifications for Surry Power Station Units No. 1 and 2.

On this same day, representatives from Vepco and the Regulatory staff met to discuss the power distribution limits used in our ECCS evaluation and delineated in the proposed technical specifications. In compliance with a request made by the staff, the following attachments are presented as further justification for the proposed power distribution limits.

Please note that the attached material contains information that is proprietary to the Westinghouse Electric Corporation. The proprietary information has been marked by means of brackets. The basis for marking the material proprietary is identified by marginal notes referring to the standards in Section 8 of the affidavit of R. A. Wiesemann of record "In the Matter of Acceptance Criteria for Emergency Core Cooling Systems for Light Water Cooled Nuclear Power Reactors (Docket No. RM-50-1)" at transcript pages 3706 through 3710 (February 24, 1972).

Due to the proprietary nature of this material which was obtained at considerable expense to the Westinghouse Electric Corporation and the release of which would seriously affect their competitive position, we request this information to be withheld from public disclosure in accordance with the Rules of Practice, 10 CFR 2.790, and that the information presented therein be safeguarded in accordance with 10 CFR 2.903. We believe that withholding this information will not adversely affect the public interest.

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①

This information is for your internal use only and should not be released to persons or organizations outside the Office of Nuclear Reactor Regulation and the ACRS without prior approval of Westinghouse Electric Corporation. Should it become necessary to release this information to such persons as part of the review procedure, please contact Westinghouse Electric Corporation and they will make the necessary arrangements required to protect their proprietary interests.

If you have any questions or comments, please contact us.

Very truly yours,

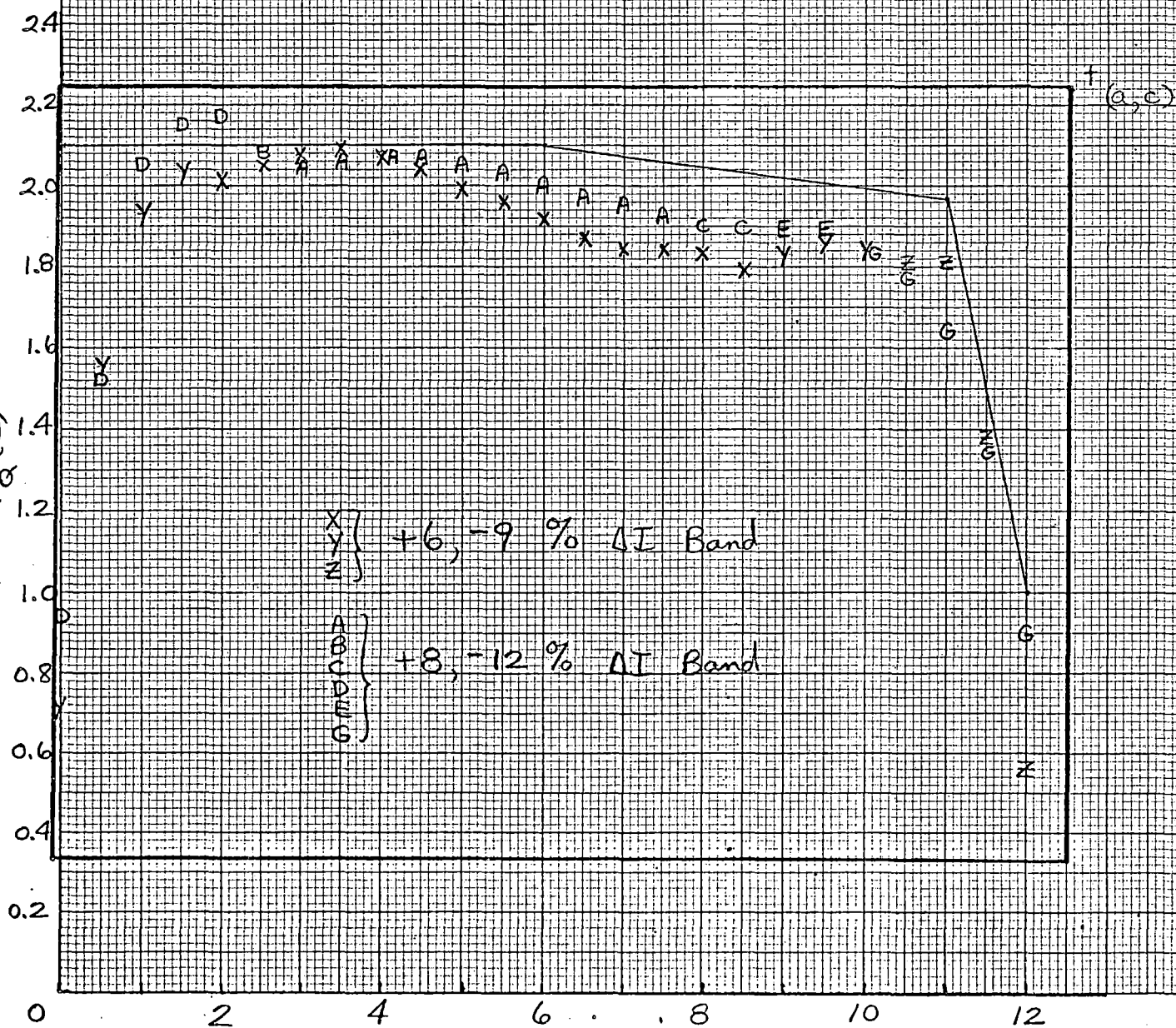


C. M. Stallings

Vice President - Power Supply
and Production Operations

Attachments

Surry 1, Cycle 2 Maximum Peaking Factor vs. Axial Core Height



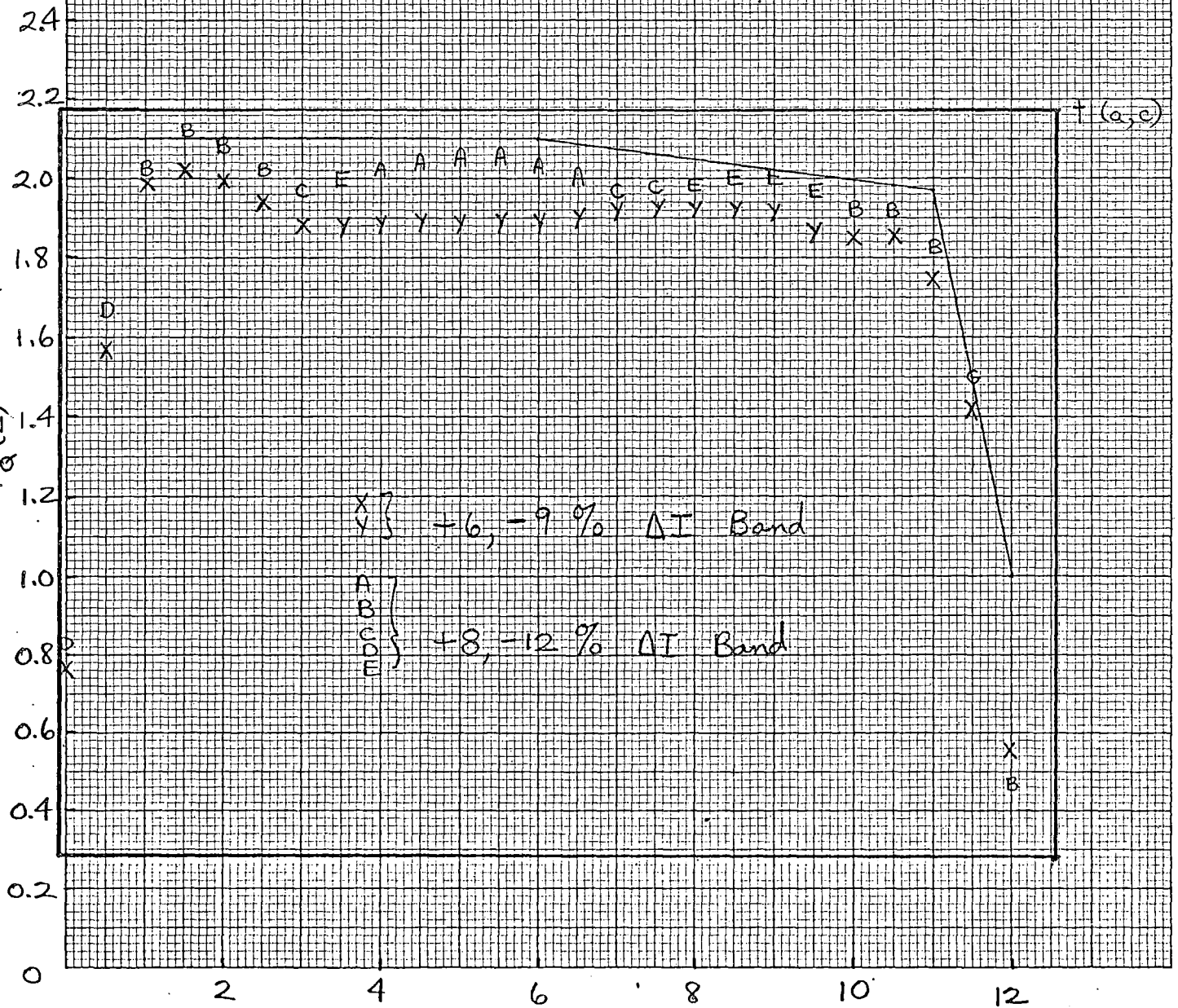
46 1320

10 X 10 TO 1/2 INCH 7 X 10 INCHES
KEUFFEL & ESSER CO. MADE IN U.S.A.

Surry 2, Cycle 2 Maximum Peaking Factor vs. Axial Core Height

46 1320

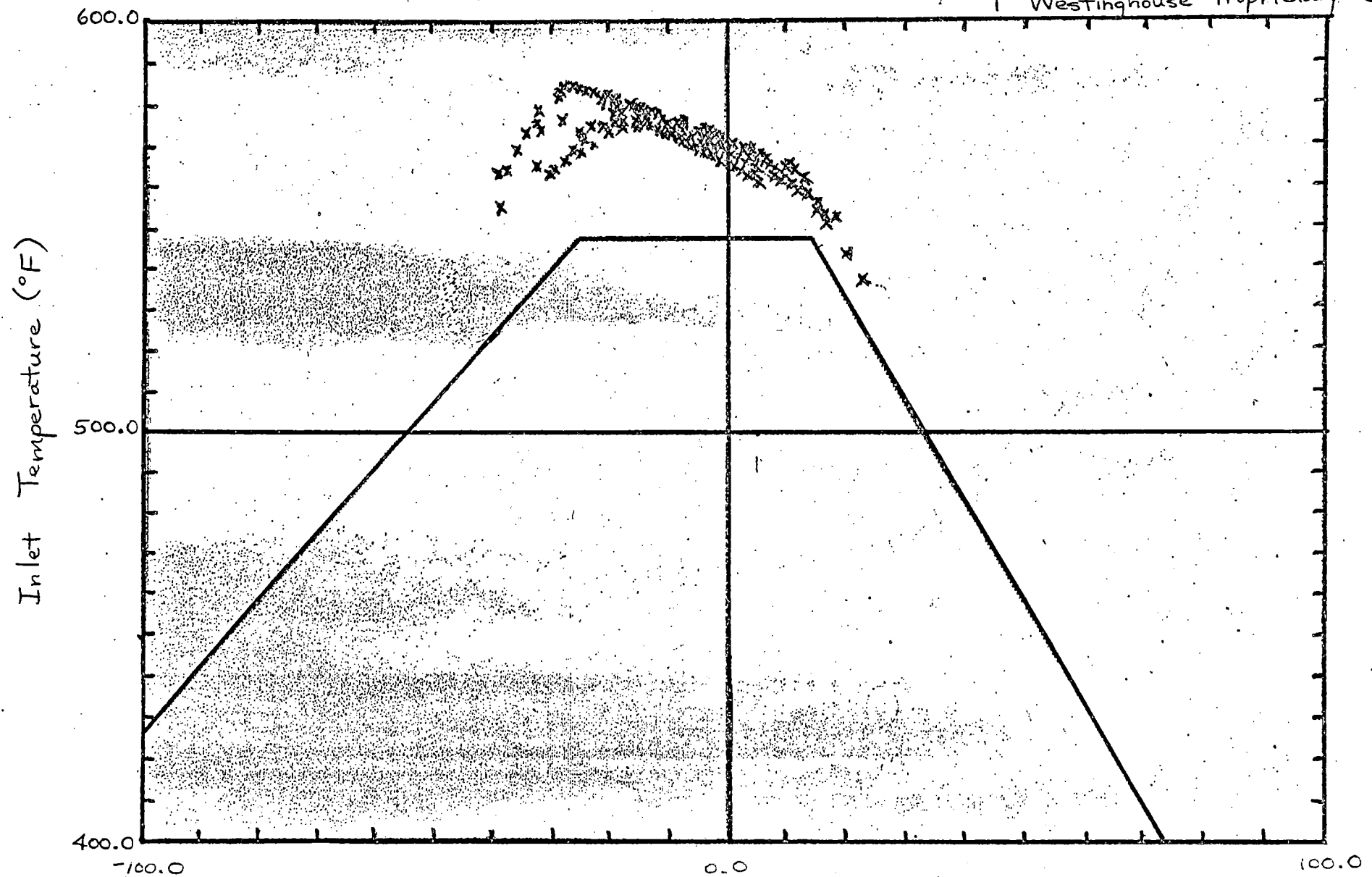
10 X 10 TO 1/4 INCH. 7 X 10 INCHES
KEUFFEL & ESSER CO. MADE IN U.S.A.



Key For Plot of $F_Q(z)$ vs Core Height

Case Type	Transient	Case I.D.	t(a,c)
1 (BOL)	100-50-100	A	
4 (EOL Base Loaded)	100-50-100	B	
1 (BOL)	100-70-100	C	
4 (EOL Base Loaded)	100-70-100	D	
1 (BOL)	100-30-100	E	
4 (EOL Base Loaded)	100-30-100	G	
7 (EOL Load Following)	100-50-100	X	
1 (BOL)	100-50-100	Y	
4 (EOL Base Loaded)	100-50-100	Z	

† Westinghouse Proprietary Class 2



Axial Offset (Percent)

Surry 2 / +8, -12 / 100-50-100 / NO PL / 118 PERCENT POWER †(a,c)