

Integrated Action Plan For Digital I&C and Future Regulatory Modernization - Panel -

11th Nuclear Plant Instrumentation, Control and Human-Machine Interface Technologies

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Integrated Action Plan to Modernize Digital Instrumentation and Controls Regulatory Infrastructure

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NUCLEAR REGULATORY COMMISSION

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Enclosure

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**NUCLEAR REGULATORY ISSUE SUMMARY 2002-22, SUPPLEMENT 1,
CLARIFICATION ON ENDORSEMENT OF NUCLEAR ENERGY INSTITUTE GUIDANCE IN
DESIGNING DIGITAL UPGRADES IN INSTRUMENTATION AND CONTROL SYSTEMS.**

ADDRESSEES

All holders of operating licenses under Title 10 of the Code of Federal Regulations (10 CFR) Part 50, "Domestic Licensing of Production and Utilization Facilities."
All holders of combined licenses under 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants."

INTENT

The U.S. Nuclear Regulatory Commission (NRC) is issuing a supplement to Regulatory Issue Summary (RIS) 2002-22, "Use of EPR(N) Joint Task Force Report, 'Guidance on Licensing Digital Upgrades,' EPRC TR-02248, Revision 1, NEI 01-01, 'A Revision of EPRC TR-02248 To Reflect Changes to the 10 CFR 50.59 Rule,'" dated November 20, 2002 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML02044044). RIS 2002-22 endorses Nuclear Energy Institute (NEI) 01-01, "Guidance on Licensing Digital Upgrades," EPRC TR-02248, Revision 1, NEI 01-01, "A Revision of EPRC TR-02248 To Reflect Changes to the 10 CFR 50.59 Rule," issued March 2002 (ADAMS Accession No. ML02000169). NEI 01-01 provides guidance for designing, licensing, and implementing digital upgrades and replacements to instrumentation and control (I&C) systems (hereinafter referred to as "digital I&C") in a consistent and comprehensive manner.

The RIS supplement clarifies RIS 2002-22, which remains in effect. The NRC continues to endorse NEI 01-01 as stated in RIS 2002-22, as clarified by this RIS supplement. Specifically, the guidance in this RIS supplement clarifies the NRC staff's endorsement of the guidance pertaining to NEI 01-01, Sections 4 and 5 and Appendices A and B. This RIS supplement clarifies the guidance for preparing and documenting "justification assessments." Just can be used to evaluate the likelihood of failure of a proposed digital modification, including the likelihood of failure due to a common cause (i.e., a common-cause failure (CCF)). Licensees can use these qualitative assessments to ensure a condition that a proposed digital I&C modification has a sufficiently low "likelihood of failure." This conclusion and the reasons for it should be documented, as required by 10 CFR 50.59(a)(5), as part of the evaluations of proposed digital modifications.

BACKGROUND

The nuclear power industry continues to replace aging instrumentation and control (I&C) systems with modern I&C systems. I&C technologies provide increased reliability and safety benefits, but can also introduce new types of potential CCF hazards. For example, software design errors, programming errors, or hardware design errors could result in a CCF of redundant trains with identical I&C systems.

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¹ On page 4-2 of NEI 01-01, NEI defines "sufficiently low" to mean much lower than the likelihood of failure that are introduced in the system that would produce more CCF (RIS) (i.e., single failure and comparable to other CCFs that are not considered in the USFAR) (i.e., design basis, maintenance errors, vibration errors).

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U.S. NUCLEAR REGULATORY COMMISSION

**DESIGN-SPECIFIC REVIEW STANDARD
for NuScale SMR DESIGN**

7.0 INSTRUMENTATION AND CONTROLS—INTRODUCTION AND OVERVIEW OF REVIEW PROCESS

This Design Specific Review Standard (DSRS) section provides guidance to the staff of the U.S. Nuclear Regulatory Commission (NRC) to use in reviewing the instrumentation and control (I&C) design of the NuScale Power (NuScale) modular nuclear reactor (SMR) nuclear power reactor. This guidance will help the staff in determining whether the design complies with the applicable regulatory requirements and whether the applicant has demonstrated that there is reasonable assurance that the design will adequately protect public health and safety. This DSRS was developed as a guide for the NuScale SMR design, and is not applicable to other designs unless specifically addressed in DSRS documents for that design center because the guidance focuses on NuScale SMR design-specific technical matters.

Major Differences between the DSRS and the Standard Review Plan

The guidance in this DSRS chapter differs from the guidance in Chapter 7 of the Standard Review Plan (SRP) (NUREG-080, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants, LWR Edition," issued in 2007). This DSRS chapter reflects some important lessons the staff learned when using the SRP to review new large light water reactor (LWR) designs.

The staff has incorporated the following lessons learned into this guidance:

- This guidance emphasizes fundamental I&C design principles of independence, redundancy, predictability and repeatability, and diversity and defense-in-depth (DID). The staff intends to verify an applicant has shown the instrumentation and control (I&C) design incorporates these principles through analysis, such as hazard analysis. These principles are cornerstones of the staff's review in this area. The current SRP guidance is system-focused and does not take advantage of such a unified framework. This guidance aims to address all the significant aspects of the I&C design in a unified manner through the framework.
- This guidance highlights only those I&C regulatory requirements and guidance applicable to the NuScale SMR. The existing SRP discusses regulatory requirements that are not applicable to the design of digital I&C systems as governed by the legal requirements set forth in NRC regulations (10 CFR Part 50, Appendix A and 10 CFR 50.59(a)), which incorporate by reference tables of technical specifications (TS) and safety analysis reports (SAR) for LWR reactors, and these I&C standards, as well as 10 CFR 50.59(a), which address some of the unique system, operational, maintenance, design, and other "requirements." These items are well-understood in the I&C technical community, and are not "requirements" in the DSRS. The DSRS focuses on the unique requirements for SMR design that are not covered in the NRC regulations that are not covered in the DSRS. These "requirements" are addressed in the DSRS, and are not "requirements" in the DSRS. The DSRS focuses on the unique requirements for SMR design that are not covered in the NRC regulations that are not covered in the DSRS. These "requirements" are addressed in the DSRS, and are not "requirements" in the DSRS.

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Panel Discussion

- Opening Remarks
- Panel Discussion
- Audience Q & A

Panelists

- Ray Herb – Southern Nuclear
- Neil Archambo – Duke Energy
- Brian Gardes – NuScale Power
- Mark Burzynski – SunPort SA

NRC DI&C Integrated Action Plan (IAP)

(SECY-16-0070)

- Improve the regulatory infrastructure for Digital I&C
- Consider the broad context of DI&C regulatory challenges
- Address areas that have substantial near-term benefit
 - Treatment of Potential DI&C Common Cause Failure
 - Use of 50.59 for Digital Upgrades
 - Commercial Grade Dedication
 - Earlier Licensing Approval
- Consider long-term improvements to address broader regulatory challenges and evolving technologies

IAP Modernization Plans (MP)

- MP #1 Protection against Common Cause Failure (CCF)
- MP #2 Guidance for 10 CFR 50.59
- MP #3 Acceptance of Digital Equipment
- MP #4 Assessment for Modernizing I&C Regulatory Infrastructure

2018 Accomplishments

- Review of NRC CCF Policy (SECY-18-0060)
- NRC Guidance for Qualitatively Assessing Likelihood of Common Cause Failure in 50.59 Upgrades (Supplement 1 to RIS 2002-22)
- Industry Guidance for 50.59 Upgrades (Appendix D to NEI 96-07)
- NRC Guidance for Early Licensing of Major Digital Systems (DI&C ISG-06)
- Design Specific Review Guidance (NuScale DSRS)

NRC Guidance Activities in 2019

- Update to BTP 7-19 for Addressing Diversity and Defense in Depth for CCF
- Evaluation of IEC Standards as Alternatives to Meet the IEEE 279/603 Criteria in 10 CFR 50.55a(h)
- Endorsement Review of Commercial Grade Dedication Guidance
- Advanced Reactor Design Review Guidance
- Broad Assessment of Regulatory Infrastructure

Panel Discussion

- What are emergent challenges within the various digital I&C Communities?
 - U.S. Operating Reactor Fleet
 - New and Advanced Reactors
 - Research and Test Reactors
 - Digital I&C Vendors
 - International
- What are key industry guidance and research development initiatives?
- What are broader opportunities to transform the NRC regulatory framework?