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52-026ND-19-0184  
10 CFR 52.99(c)(3)U.S. Nuclear Regulatory Commission  
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Washington, DC 20555-0001

Southern Nuclear Operating Company  
Vogtle Electric Generating Plant Unit 3 and Unit 4  
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load  
Item 3.3.00.02a.ii.b [Index Number 765]

Ladies and Gentlemen:

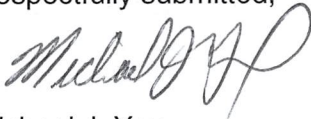
Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of February 21, 2019, Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Uncompleted Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 3.3.00.02a.ii.b [Index Number 765] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing this ITAAC. Southern Nuclear Operating Company will, at a later date, provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI 08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact Tom Petrak at 706-848-1575.

Respectfully submitted,



Michael J. Yox  
Regulatory Affairs Director Vogtle 3 & 4

Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4  
Completion Plan for Uncompleted ITAAC 3.3.00.02a.ii.b [Index Number 765]

MJY/PGL/sfr

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**Southern Nuclear Operating Company  
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Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4  
Completion Plan for Uncompleted ITAAC 3.3.00.02a.ii.b [Index Number 765]**

## **ITAAC Statement**

### **Design Commitment**

2.a) The nuclear island structures, including the critical sections listed in Table 3.3-7, are seismic Category I and are designed and constructed to withstand design basis loads as specified in the Design Description, without loss of structural integrity and the safety-related functions.

### **Inspections/Tests/Analyses**

ii) An inspection of the as-built concrete thickness will be performed.

### **Acceptance Criteria**

ii.b) A report exists that concludes that the as-built concrete thicknesses of the shield building sections conform to the building sections defined in Table 3.3-1.

## **ITAAC Completion Description**

Multiple ITAAC are performed to verify the nuclear island structures, including the critical sections listed in Combined License (COL) Appendix C, Table 3.3-7 (Attachment A), are seismic Category I and are designed and constructed to withstand design basis loads as specified in the Design Description, without loss of structural integrity and the safety-related functions. The subject ITAAC requires that an inspection be performed and documented in a report that concludes that the as-built shield building sections concrete wall thicknesses conform to the building sections defined in COL Appendix C, Table 3.3-1 (Attachment B).

The inspections are performed of the as-built sections (following concrete placement) in accordance with the requirements of measurement guideline APP-GW-IT-001 (Reference 1), which identifies the location and frequency of inspection points for determining wall thickness to ensure the resulting measurements are representative of the entire section being inspected. The measurements are based on the size and construction type of each section. Measurements are taken using survey equipment in accordance with site survey procedures (Reference 2).

The inspection results contained in the Unit 3 and Unit 4 principal closure documents (References 3 and 4, respectively) and summarized in Attachment B conclude that the as-built concrete thicknesses of the shield building sections conform to the building sections defined in ITAAC Table 3.3-1.

References 1 thru 4 are available for NRC inspection as part of the Unit 3 and Unit 4 ITAAC 3.3.00.02a.ii.b Completion Packages (Reference 5 and 6, respectively).

## **List of ITAAC Findings**

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and

associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

**References (available for NRC inspection)**

1. APP-GW-IT-001, Revision 0, "Guidelines for Concrete Wall and Slab Thickness Measurements"
2. 26139-000-4MP-T81C-N3201, Revision 4, "Construction Survey"
3. Principal Closure Document (Unit 3)
4. Principal Closure Document (Unit 4)
5. 3.3.00.02a.ii.b-U3-CP-Rev0, ITAAC Completion Package
6. 3.3.00.02a.ii.b-U4-CP-Rev0, ITAAC Completion Package
7. NEI 08-01, "Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52"

**Attachment A**

Excerpt from COL Appendix C, Table 3.3-7\*  
Nuclear Island Critical Structural Sections

Auxiliary and Shield Building\*

Shield building roof, exterior wall of the PCS water storage tank

Shield building roof, interior wall of the PCS water storage tank

Shield building roof, tension ring and air inlets

Shield building SC cylinder

Shield building SC to RC connection

\* Excerpt from COL Appendix C, Table 3.3-7

**Attachment B**

Excerpt from COL Appendix C, Table 3.3-1\*  
 Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building<sup>(1)</sup>

Wall or Section Description*	Column Lines* (7)	Floor Elevation or Elevation Range* (7)(8)	Concrete Thickness* (2)(3)(4)(5)(9)	Inspection results +	
				Minimum recorded thickness	Maximum recorded thickness
<b>Shield Building* (6)</b>					
Shield Building Cylinder	Not Applicable	From 100'-0" to 248'-6"	3'-0" (including 3/4 inch thick min. steel plate liner on each face on portion not protected by auxiliary building)	X'-xx.x"	Y'-yy.y"



Wall or Section Description*	Column Lines* (7)	Floor Elevation or Elevation Range* (7)(8)	Concrete Thickness* (2)(3)(4)(5)(9)	Inspection results +	
				Minimum recorded thickness	Maximum recorded thickness
<b>Shield Building* (6)</b>					
Air Inlet	Not Applicable	From 248'-6" to 251'-6"	3'-0" (including 3/4 inch thick min. steel plate liner on each face)	X'-xx.x"	Y'-yy.y"
		From 251'-6" to 254'-6"	3'-0" to 4'-6" (including 1 inch thick steel plate liner on each face)	X'-xx.x"	Y'-yy.y"
		From 254'-6" to 266'-4"	4'-6" (including 1 inch thick min. steel plate liner on each face)	X'-xx.x"	Y'-yy.y"
Tension Ring	Not Applicable	From 266'-4" to 271'-6" (at top of plate)	4'-6" (including 1-1/2 inch thick steel plate liner on each face)	X'-xx.x"	Y'-yy.y"

Wall or Section Description*	Column Lines* (7)	Floor Elevation or Elevation Range* (7)(8)	Concrete Thickness* (2)(3)(4)(5)(9)	Inspection results +	
				Minimum recorded thickness	Maximum recorded thickness
<b>Shield Building* (6)</b>					
Conical Roof	Not Applicable	From 271'-6" to 293'-9"	3'-0" (including 1/2 inch thick min. steel plate liner on bottom face, outside of PCS tank exterior wall)	X'-xx.x"	Y'-yy.y"
PCS Tank External Cylindrical Wall	Not Applicable	From 293'-9" to 328'-9"	2'-0"	X'-xx.x"	Y'-yy.y"
PCS Tank Internal Cylindrical Wall	Not Applicable	From 309'-4" to 329'-0"	1'-6"	X'-xx.x"	Y'-yy.y"
PCS Tank Roof	Not Applicable	328'-9" (Lowest) 329'-0" (Highest)	1'-3"	X'-xx.x"	Y'-yy.y"
Nuclear Island Basemat	Below shield building	From 60'-6" to containment vessel or 82'-6"	6'-0" <sup>(10)</sup> to 22'-0" (varies)	X'-xx.x"	Y'-yy.y"

Notes:

+ Inspection results are Unit specific

\* Excerpt from COL Appendix C, Table 3.3-1

1. The column lines and floor elevations are identified and included on Figures 3.3-1 through 3.3-13.
2. These wall (and floor) thicknesses have a construction tolerance of  $\pm 1$  inch, except as noted and for exterior walls below grade where the tolerance is +12 inches, - 1 inch. These tolerances are not applicable to the nuclear island basemat.
3. For walls that are part of structural modules, the concrete thickness also includes the steel face plates. Where faceplates with a nominal thickness of 0.5 inches are used in the construction of the wall modules, the wall thicknesses in this column apply. Where faceplates thicker than the nominal 0.5 inches are used in the construction of the structural wall modules, the wall thicknesses in the area of the thicker faceplates are greater than indicated in this column by the amount of faceplate thickness increase over the nominal 0.5 inches. Overlay plates are not considered part of the faceplates, and thus are not considered in the wall thicknesses identified in this column.
4. For floors with steel surface plates, the concrete thickness also includes the plate thickness.
5. Where a wall (or a floor) has openings, the concrete thickness does not apply at the opening.
6. The elevation ranges for the shield building items are rounded to the nearest inch.
7. The Wall or Section Description, Column Line information, and Floor Elevation or Elevation Ranges are provided as reference points to define the general location. The concrete thickness of an item intersecting other walls, roofs or floors at a designated location (e.g., column line) is not intended to be measured to the stated column line, but only to the point where the intersection occurs.
8. Where applicable, the upper wall portions extend to their associated roofs, which may vary in elevation, e.g., sloped roofs.
9. From one wall/floor section to another, the concrete thickness transitions from one thickness to another, consistent with the configurations in Figures 3.3-1 through 3.3-14.
10. The 6-foot concrete thickness includes the thickness of the containment vessel bottom head in a local area at the center of containment.