

NRR-DMPSPEm Resource

From: Wengert, Thomas
Sent: Wednesday, February 27, 2019 2:15 PM
To: Arnold, Timothy
Cc: BICE, DAVID B (ANO); Pyle, Stephenie; Pascarelli, Robert
Subject: ANO-1 - Final RAI RE: Relief Request ANO1-ISI-032 (EPID L-2018-LLR-0135)
Attachments: ANO-1 Final RAI for Relief Request ANO1-ISI-032.pdf

On February 21, 2019, the U.S. Nuclear Regulatory Commission (NRC) staff sent Entergy Operations, Inc. (Entergy) the draft Request for Additional Information (RAI) identified below. This RAI relates to relief request ANO1-ISI-032, concerning a visual examination of the reactor vessel support performed during the fourth inservice inspection (ISI) interval at Arkansas Nuclear One, Unit 1 (ANO-1).

On February 27, 2019, the NRC staff had a conference call with Entergy to clarify the request. At the conclusion of the call, the licensee agreed to provide a response to this RAI within 30 days of this correspondence. A publicly available version of this final RAI (attached with "Draft" removed) will be placed in the NRC's Agencywide Documents Access and Management System (ADAMS).

From: Wengert, Thomas
Sent: Thursday, February 21, 2019 10:03 AM
To: Arnold, Timothy
Cc: BICE, DAVID B (ANO) ; 'Halter, Mandy' ; Pyle, Stephenie
Subject: ANO-1 - Draft RAI RE: Relief Request ANO1-ISI-032 (EPID L-2018-LLR-0135)

Mr. Arnold,

By letter dated October 31, 2018 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML18304A450), Entergy Operations, Inc. (Entergy, the licensee) requested relief from certain requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. This relief request, ANO1-ISI-032, pertains to an impracticality determination of visual examination of the reactor vessel support performed during the fourth inservice inspection (ISI) interval, the third inspection period, at Arkansas Nuclear One, Unit 1 (ANO-1).

The U.S. Nuclear Regulatory Commission (NRC) staff has determined that additional information, as described in the attached request for additional information (RAI), is required for the staff to complete its review of this relief request. This RAI is identified as draft at this time to confirm your understanding of the information that the NRC staff needs to complete the evaluation. If the request for information is understood, please respond to this RAI within 30 days of the date of this request. Please contact me if you would like to set up a conference call to clarify this request for information.

Tom Wengert
U.S. Nuclear Regulatory Commission
Project Manager – Arkansas Nuclear One
NRR/DORL/LPL4
(301) 415-4037

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Email Number: 835

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Post Office:

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REQUEST FOR ADDITIONAL INFORMATION

RELIEF REQUEST ANO1-ISI-032

REGARDING EXAMINATION COVERAGE FOR

SUPPORTS OTHER THAN PIPING SUPPORTS

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT 1

DOCKET NO. 50-313

By letter dated October 31, 2018 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML18304A450), Entergy Operations, Inc. (Entergy, the licensee) requested relief from certain requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. This relief request, ANO1-ISI-032, pertains to an impracticality determination of visual examination of the reactor vessel support performed during the fourth inservice inspection (ISI) interval, the third inspection period, at Arkansas Nuclear One, Unit 1 (ANO-1).

The U.S. Nuclear Regulatory Commission staff reviewed the relief request and needs additional information to complete the review. The staff's requests for additional information (RAIs) are listed below:

RAI-1

In Relief Request ANO1-ISI-032, the licensee states, in part, that Entergy will continue to perform pressure testing on the subject components as required by the ASME Code, Section XI. Since reactor vessel supports are not pressure-retaining components, explain how the inspection results from pressure testing can provide useful information concerning the integrity of the reactor vessel support.

RAI-2

For the reactor vessel skirt circumferential weld (formerly Component 01-032), the relief request states that it was not accessible, implying that the associated coverage is 0 percent, the same as what had been achieved in the last inspection of this weld for the third ISI interval (ADAMS Accession No. ML101170119). However, the figure on the last page of the supplement (ADAMS Accession No. ML093070060) supporting the previous relief request for the third ISI interval seems to suggest that if the fiber scope can be directed to go through the 2-inch hole adjacent to the circumferential weld and the viewing angle of the scope can be remotely adjusted, then certain coverage of the skirt circumferential weld could be obtained. Please confirm that improvements can be made in the next inspection by using a more flexible fiber scope conduit and a viewing angle adjustable head, or provide details to explain the inaccessibility of the weld by the fiber scope using the available devices.