Exelon Generation®

Dresden Nuclear Power Station
6500 North Dresden Road
Morris, IL 60450

February 8, 2019

SVPLTR #19-0005

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Dresden Nuclear Power Station, Unit 3
Renewed Facility Operating License No. DPR-25
NRC Docket No. 50-249

Subject: Owner's Activity Report Submittal
Fifth 10-Year Interval 2018 Refueling Outage Activities

This letter submits the Owner's Activity Report (i.e., Form OAR-1) and In Vessel Visual Inspection (IVVI) Report for the Dresden Nuclear Power Station (DNPS) Unit 3 refueling outage (D3R25) which began on October 29, 2018 and was completed on November 12, 2018. This is the second refueling outage conducted in the second inspection period of the fifth 10-year inservice inspection interval for DNPS Unit 3. A copy of the Owner's Activity Report and IVVI Report are provided as attachments to this letter.

This Owner's Activity Report is submitted in accordance with American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Case N-532-4, "Repair/Replacement Activity Documentation Requirements and Inservice Summary Report Preparation and Submission," and US NRC Safety Evaluation in support of request for relief associated with the fifth 10-Year Inservice Inspection Interval Program dated September 30, 2013. Code Case N-532-4 requires an Owner's Activity Report Form OAR-1 to be prepared and certified upon completion of each refueling outage. In accordance with the conditions of Code Case N-532-4, this OAR-1 form is being submitted within 90-days of the completion of the refueling outage.

The IVVI results are provided to report vessel internal inspections and to support B-N-1 and B-N-2 relief request exam completion. Should you have any questions concerning this letter, please contact Mr. Bruce Franzen, Regulatory Assurance Manager at (815) 416-2800.

Respectfully,

Peter J. Karaba
Site Vice President
Dresden Nuclear Power Station

Enclosures

A047
NRR

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number: Refueling Outage D3R25 OAR-1

Plant: Dresden Nuclear Power Station, 6500 N. Dresden Road, Morris, IL 60450

Unit No. 3 (if applicable) Commercial Service Date: 11/16/1971 Refueling Outage Number: D3R25

Current Inspection Interval: 5th Inspection Interval (1st, 2nd, 3rd, 4th, other)

Current Inspection Period: 2nd Inspection Period (1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the Inspection Plans: 2007 Edition with 2008 Addenda

Date / Revision of Inspection Plans: 1/26/18/Revision 4

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans: N/A

Code Cases used: N-649, N-661-2, N-586-1

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests, meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of D3R25 (refueling outage number) conform to the requirements of Section XI

Signed John N. Kish, ISI Coordinator (Owner or Owner's designee. Title) Date 2/1/19

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, Connecticut

have inspected the items described in this Owner's Activity Report, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI

By signing this certificate neither the Inspector nor his employer makes any warranty expressed or implied concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection

(Inspector's Signature) Commissions: IL 1546; NB10972 A, N, I (National Board, State, Province, and Endorsements)

Date 2/6/19

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category and Item Number	Item Description	Evaluation Description
C-B/C2.21	Unacceptable Flaw Indications Found in Isolation Condenser Nozzle to Shell Weld	SIA Calculation 1801382.301

TABLE 2
ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Item Description	Description Of Work	Date Completed	Repair/ Replacement Plan Number
2/3	3A LPCI Heat Exchanger Support Repairs	Removed Indications and Re-Welded Support	11/6/18	RRP 3-18-051
2/3	3B LPCI Heat Exchanger Support Repairs	Removed Indications and Re-Welded Support	11/7/18	RRP 3-18-052
3	Repair of Thru Wall CCSW Leak	Overlaid Thru Wall Leak	4/11/17	RRP 3-17-008

Dresden Unit 3
Reactor Internals Inspection Report
Refueling Outage D3R25

The ASME Section XI inspections credited during D3R25 includes the once-per-period B-N-1 inspection of the reactor vessel interior and B-N-2 inspections of reactor vessel interior attachments. Credit is being taken for the Boiling Water Reactor Vessel and Internals Project (BWRVIP) examinations in place of the B-N-1 and B-N-2 examinations as allowed in Dresden Station Relief Request I5R-07, "Request for Relief for the Use of BWRVIP Guidelines in Lieu of Specific ASME Code Requirements on Reactor Pressure Vessel Internals and Components Inspection In Accordance with 10CFR50.55a(a)(3)(i) Alternative Provides Acceptable Level of Quality and Safety."

To implement the requirements of the BWRVIP, General Electric – Hitachi (GEH) was contacted to perform the inspection activities. The following components and assemblies were examined as directed by BWRVIP documents:

- Six core spray piping welds, all four core spray lower sectional replacements and two core spray piping brackets in accordance with the BWRVIP-18 Revision 2-A, "BWR Core Spray Internals Inspection and Flaw Evaluation Guidelines."
- Six shroud/shroud support vertical welds and two components on the shroud repair hardware in accordance with BWRVIP-76 Revision 1-A, "BWR Core Shroud Inspection and Flaw Evaluation Guidelines."
- Two shroud support horizontal welds in accordance with BWRVIP-38, "BWR Shroud Support Inspection and Flaw Evaluation Guidelines."
- Attachment welds for two core spray piping brackets, two separator guide rod brackets, four dryer guide rod brackets, twelve surveillance sample holder brackets and four steam dryer support lugs in accordance with BWRVIP-48-A, "Vessel ID Attachment Weld Inspection and Flaw Evaluation Guidelines."
- The top guide aligner pin/block assemblies at all four locations in accordance with BWRVIP-26-A, "BWR Top Guide Inspection and Flaw Evaluation Guidelines."

The above BWRVIP examinations resulted in no indications identified in the reactor interior surface as defined by B-N-1 or the reactor interior attachments as defined by B-N-2.