hgulatory Docket File

Frederick W. Schneider Vice President Production Public Service Electric and Gas Company 80 Park Place Newark, N.J. 07101 201/622-7000

January 4, 1977

Mr. James P. O'Reilly Director of USNRC Office of Inspections and Enforcements Region 1 631 Park Avenue King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly:

LICENSE NO. DPR-70 DOCKET NO. 50-272 REPORTABLE OCCURRENCE 76-17/3L

Pursuant to the requirements of Salem Generating Station Unit No. 1 Technical Specifications, Section 6.9.1, we are submitting Licensee Event Report for Reportable Occurrence 76-17/3L. This report is required within thirty (30) days of the occurrence.

Sincerely yours,

CC: Director, Office of Inspection and Enforcement (30 copies)
Director, Office of Management
Information and Program Control
(3 copies)



Report Number:

76-17/3L

Report Date:

08/76

Occurrence Date:

12/06/76

Facility

Salem Generating Station

Public Service Electric & Gas Company Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Inoperable personnel air lock - Elevation 100'.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 3 operation, Plant pressure 2235 psig, Plant temperature 532°F.

DESCRIPTION OF OCCURRENCE:

On 12/6/76, the NRC inspector, in performing routine inspections of the Operations Department surveillance procedure, identified that the acceptance value for the elevation 100' and 130' personnel air lock seal air flow was inaccurate as specified in SP(0)4.6.1.3. Subsequent investigation by the licensee of the air flow calculation indicated that the acceptance value should be < 32% versus < 65%. The Operations Department implemented Technical Specification Action Statement 3.6.1.3.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Upon entering Mode 4 on 11/24/76, the elevation 100' personnel air lock was inoperable as per Technical Specifications L.C.O. paragraph 3.6.1.3 due to a calculation error associated with the acceptance value specified in SP(O)4.6.1.3.

ANALYSIS OF OCCURRENCE:

During Mode 4 and Mode 3, the elevation 100' personnel air lock interior door exceeded the minimum acceptable leakage rate as specified in S.T.S. 3.6.1.3. At no time during this period did the containment pressure exceed that specified by Technical Specifications and at no time was there any hazard to the general public or site personnel.

CORRECTIVE ACTION:

The Maintenance Department replaced the elevation 100' personnel air lock interior door seal, replaced the latch bolts and repositioned the latching mechanism. The Operations Department issued a procedural change to reflect the correct acceptance value and retested the personnel air lock satisfactorily in accordance with SP(0)4.6.1.3.

FAILURE DATA:

This is the first failure of this type.

Component: Elevation 100' personnel air lock
Manufacturer: Chicago Bridge and Iron Company

| Prepared by B. Canfield | N/Heller |
|-------------------------|------------------------------------|
| | Manager - Salem Generating Station |
| SORC Meeting No. 122-76 | |

MIM

LICENSEE EVENT REPORT

| CONTROL BLOCK |
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| LICENSEE LICENSE EVENT TYPE TYPE |
| 回[NIJ[SIG[S]]] [0101-10101010101-1010] [41111111] [011] |
| CATEGORY TYPE SOURCE OCCNET NUMBER EVENT DATE REPORT DATE [0] CONT[P 0] [L] [L] [0 5 0 - 0 2 7 2] [1 2 0 6 7 6] [1 2 0 8 7 6] [57 52 53 55 65 51 |
| EVENT DESCRIPTION [0 2 On 12/6/76, the NRC inspector, in performing routine inspection of the |
| 7 8 9 O[3] Operations Department Surveillance Procedures, identified that the accept- |
| 7 8.9 SO SO O[4] ance value of ≤ 65% for the Elev. 100' and 130' personnel air lock seal; |
| 7 89 [0]5 air flow was inaccurate as specified in SP(0)4.6.1.3. Subsequent investi- |
| 7 8 9 90 80 gation by the licensee of the air flow calculation indicated that the |
| 7 8 9 PRIME 80 SYSTEM CAUSE COMPONENT COMPONENT COMPONENT VIOLATION |
| 07 S A D X X X X X X X X X X X X X X X X X X |
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| CAUSE DESCRIPTION [0]8 Upon entering mode four on 11/24/76, the Elev. 100' personnel air lock was |
| inoperable as per Technical Specification L.C.O. paragraph 3.6.1.3 due to |
| a calculation error in SP(0)4.6.1.3. |
| FACILITY SPOWER OTHER STATUS DISCOVERY DESCRIPTION III G 000 N/A d NRC Inspector |
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| 7 8 9 10 12 13 44 45 46 80 FORM OF ACTIVITY CONTENT RELEASE OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE 1 2 Z Z N/A N/A N/A 30 7 8 9 10 11 44 45 30 |
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EVENT DESCRIPTION (Continued)

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This is the first occurrence of this type. (76-17/3L)