

# Probabilistic Risk Assessment

# Human Reliability Analysis

## Lecture 5-2

Region I

Region II

Region III

The NRC's policy statement on probabilistic risk assessment (PRA) encourages greater use of this analysis technique to improve safety decisionmaking and improve regulatory efficiency. The NRC staff's PRA Implementation Plan describes activities now under way or planned to expand this use. These activities include, for example, providing guidance for NRC inspectors on focusing inspection resources on risk-important equipment, as well as reassessing plants with relatively high core damage frequencies for possible backfits.

Another activity under way in response to the policy statement is using PRA to support decisions to modify an individual plant's licensing basis (LB). This regulatory guide provides guidance on the use of PRA findings.

## Key Topics

- HRA importance
- General description
- Fundamental model
- Methods
- Validation
- Challenges

## Resources

- A. Kolaczowski, et al., “Good Practices for Implementing Human Reliability Analysis,” *NUREG-1792*, April 2005.
- J. Forester, et al., “Evaluation of Human Reliability Analysis Methods Against Good Practices,” *NUREG-1842*, September 2006.
- J. Forester, et al., “The International HRA Empirical Study: Lessons Learned from Comparing HRA Methods Predictions to HAMMLAB Simulator Data,” *NUREG-2127*, August 2014.
- J. Forester, et al., “The U.S. HRA Empirical Study: Assessment of HRA Method Predictions against Operating Crew Performance on a U.S. Nuclear Plant Simulator,” *NUREG-2156*, June 2016.
- A.M. Whaley, et al., “Cognitive Basis for Human Reliability Analysis,” *NUREG-2114*, January 2016.

## Other References

- A. Poucet, “Human Factors Reliability Benchmark Exercise: Synthesis Report,” *EUR 1222 EN*, Ispra Joint Research Centre, Commission of European Communities, August 1989.
- E. Lois, et al., “International HRA Empirical Study – Phase 1 Report: Description of Overall Approach and Pilot Phase Results from Comparing HRA Methods to Simulator Performance Data,” *NUREG/IA-0216, Vol. 1*, November 2009.
- A. Bye, et al., “International HRA Empirical Study – Phase 2 Report: Results from Comparing HRA Method Predictions to Simulator Data from SGTR Scenarios,” *NUREG/IA-0216, Vol. 2*, August 2011.
- V.N. Dang, et al., “International HRA Empirical Study – Phase 3 Report: Results from Comparing HRA Methods Predictions to HAMMLAB Simulator Data on LOFW Scenarios,” *NUREG/IA-0216, Vol. 3*, December 2014.
- H. Blackman, N. Siu, and A. Mosleh, *Human Reliability Models: Theoretical and Practical Challenges*, Center for Reliability Engineering, University of Maryland, College Park, MD, 1998.

## Other References

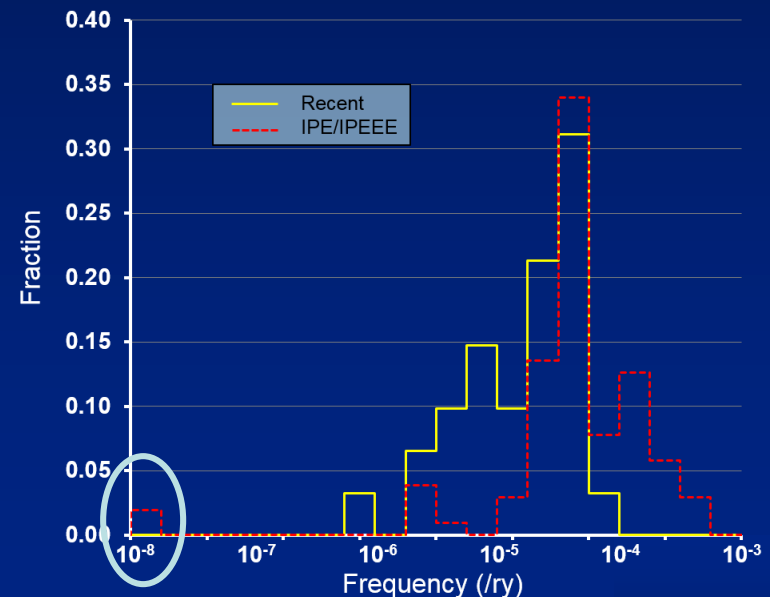
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- P. Moieni, et al., “A PC-based human reliability analysis (HRA) software,” *Proceedings ANS International Topical Meeting on Probabilistic Safety Assessment (PSA '93)*, Clearwater Beach, FL, January 26-29, 1993.
- D. Gertman, et al., “The SPAR-H Human Reliability Analysis Method,” *NUREG/CR-6883*, August 2005.
- U.S. Nuclear Regulatory Commission, “Technical Basis and Implementation Guidelines for A Technique for Human Event Analysis (ATHEANA),” *NUREG-1624, Rev. 1*, May 2000.
- J. Xing, et al., “An Integrated Human Event Analysis System (IDHEAS) for Nuclear Power Plant Internal Events At-Power Application,” *NUREG-2199, Vol. 1*, March 2017.
- Y.J. Chang and J. Xing, “The general methodology of an Integrated Human Event Analysis System (IDHEAS) for human reliability analysis method development,” *Proceedings International Conference on Probabilistic Safety Assessment and Management (PSAM 13)*, Seoul, Korea, October 2-7, 2016.
- S. Lewis and S. Cooper, “EPRI/NRC-RES Fire Human Reliability Analysis Guidelines: Final Report,” *EPRI 1023001/NUREG-1921*, July 2012.
- A. Lindeman and S. Cooper, “EPRI/NRC-RES Fire Human Reliability Analysis Guidelines – Qualitative Analysis for Main Control Room Abandonment Scenarios, Supplement 1,” *EPRI 3002009215/NUREG-1921, Supplement 1*, August 2017.

## Human Actions and NPP PRA

Operational decisions and actions have played an important role in every major NPP accident and incident

- Occurrence and progression
- Successes and failures

PRAs that don't account for human contributions are not useful (for most applications)



## Example Events

- Browns Ferry 1 & 2 cable fire (1975)
  - Worker ignites polyurethane foam, starts cable fire
  - Fire suppression delayed 7+ hours (reluctant to use water)
  - Operators achieve safe shutdown using non-safety system
- Davis-Besse loss of feedwater (1985)
  - Operator error causes loss of feedwater
  - Multiple malfunctions => feed and bleed cooling directed by procedures, would have major economic consequences
  - Shift supervisor chooses to wait for recovery of AFW (which is successful)
- Fukushima Dai-ichi Unit 1 (2011)
  - Operators close isolation condenser (little effect given accident conditions)
  - Operators perform numerous non-proceduralized actions (e.g., scavenge car batteries to supply power) in attempts to save plant
  - Ex-control room actions hampered by site conditions (tsunami alerts, aftershocks, damage, dark, radiation, ...)

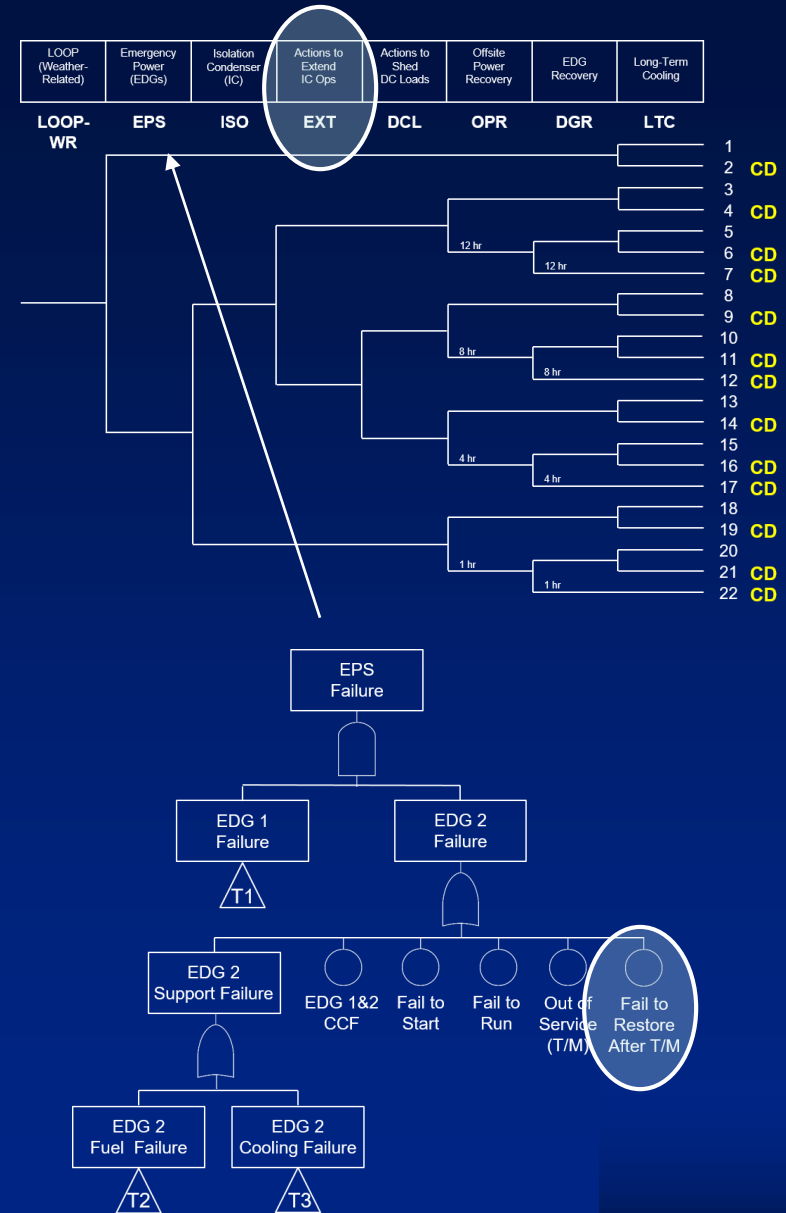
## What is HRA?

- In the context of NPP PRA: “A structured approach used to identify potential human failure events and to systematically estimate the probability of those events using data, models, or expert judgment” (NUREG-2122)
- “Human Failure Event:” interface with rest of PRA model:
  - Terminology used to emphasize connection with NPP PRA model (“basic events”), avoid connotation of blame (e.g., when time available is insufficient)
  - Includes “errors of omission,” “errors of commission”
  - Can be included at scenario level (event trees) or system level (fault trees)



# HRA General Process

- **Activities**
  - Qualitative analysis
  - Modeling
  - Quantification
- Supports overall model construction
  - Initiating event identification
  - Accident scenario modeling
  - Systems modeling
- Not just a quantification activity



# HRA Dimensions and Descriptors

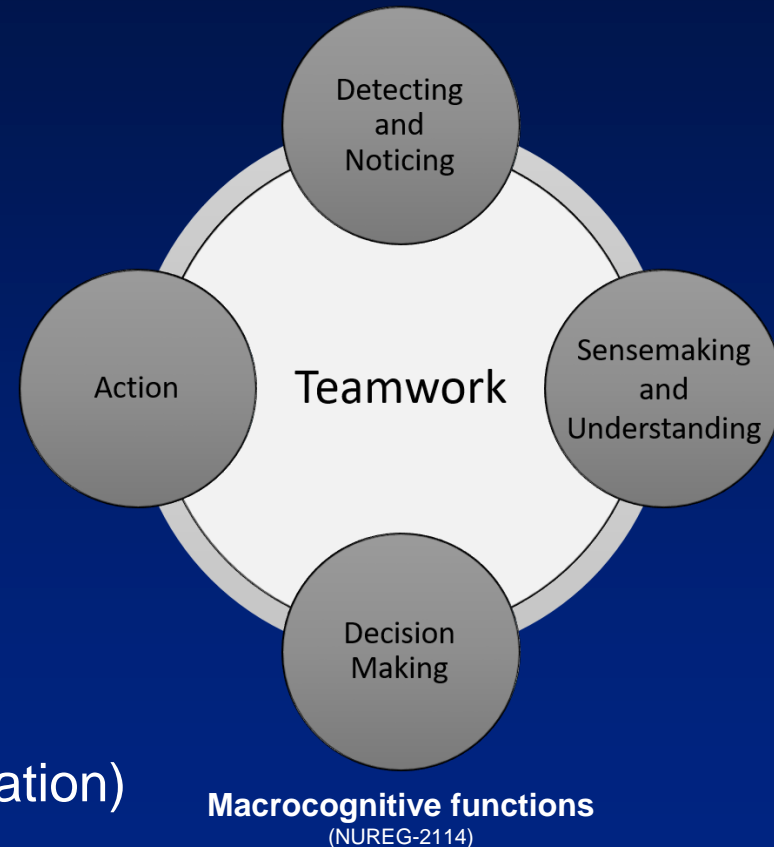
- Time
  - Pre-initiator
  - Initiator
  - Post-initiator
- Space
  - Within control room
  - Outside control room
- Organization
  - Control room crew
  - Field operators
  - Emergency response organization
- Implicit
  - Actions addressed by other PRA model elements (e.g., initiating event frequencies, loss of offsite power recovery, common cause failure probabilities)
  - Pre-initiator decisions affecting fundamental plant design (e.g., flood barrier height) and operations (e.g., resources for training)
- Out-of-scope for NPP PRA
  - Sabotage
  - Terrorism

## Typical HFE Level of Detail

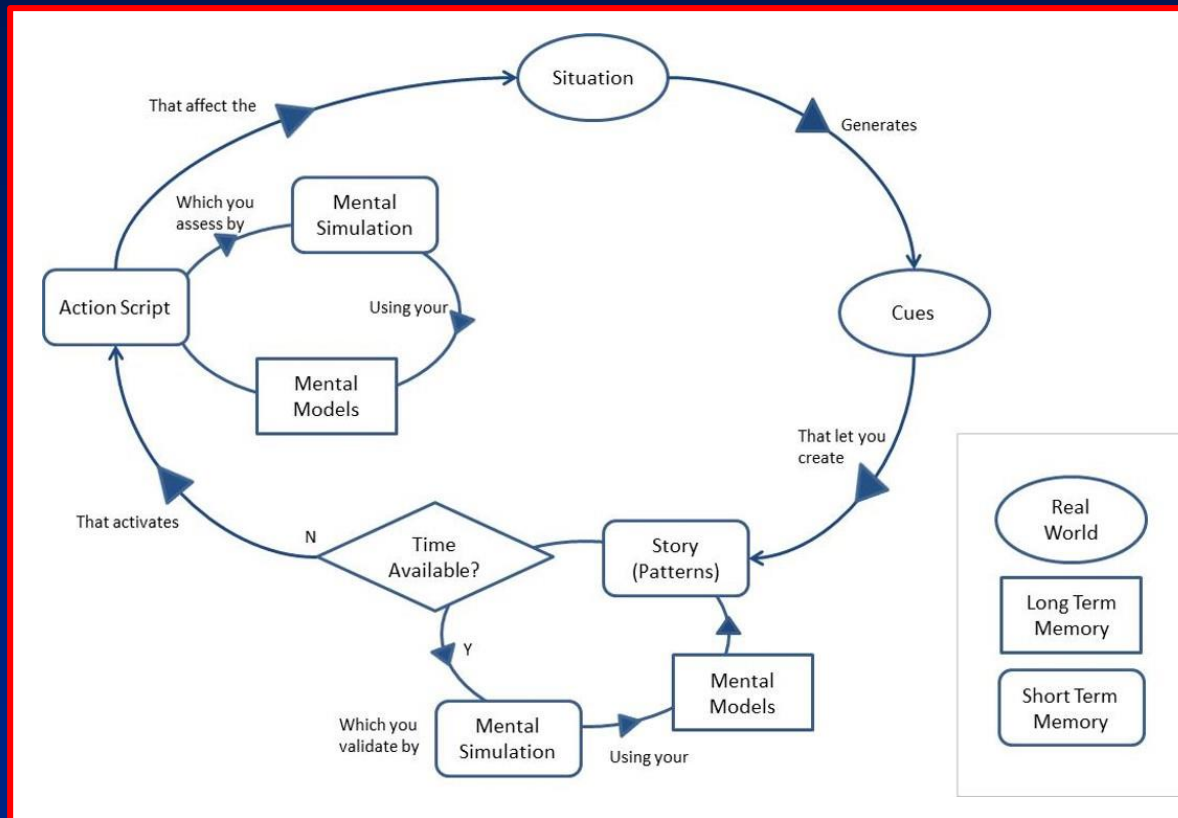
- Macro-level crew actions, e.g.,
  - Isolate faulted steam generator
  - Initiate bleed and feed cooling
  - Recover a failed pump
- Micro-level modeling (e.g., put control switch X in pull-to-lock position) can support HFE; need to consider micro-level recoveries as well as failures

## “How Things Work”

- Task-oriented view
  - Diagnosis and Planning
  - Action
- Cognitive view
  - Detecting/Noticing
  - Sensemaking/Understanding
  - Decision Making
  - Action Execution
  - Teamwork (communication/coordination)

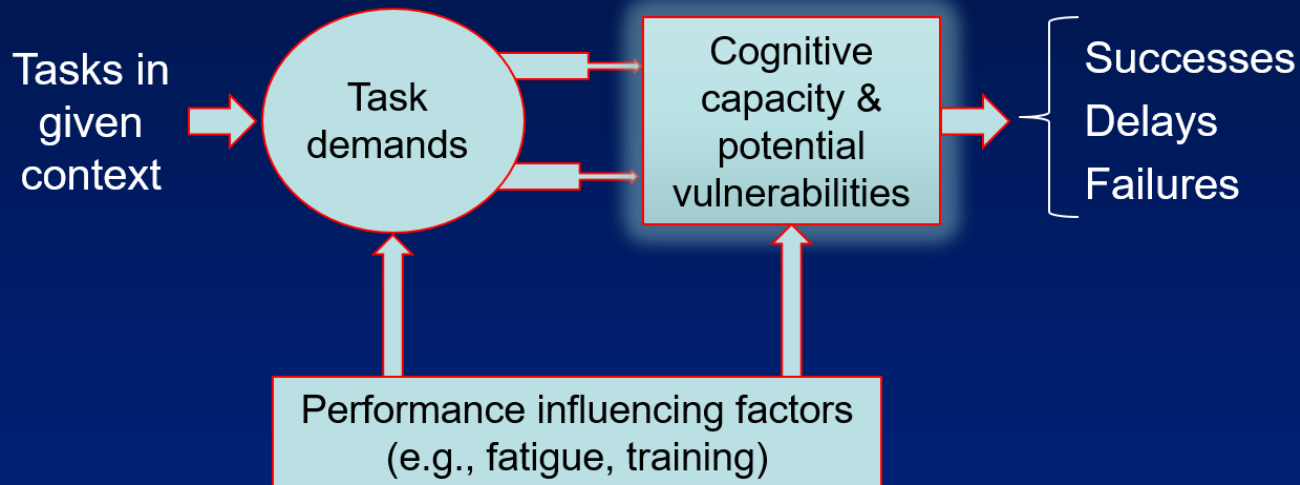


# Naturalistic Decision Making



From NUREG-2114, per F.L. Greitzer, et al., "Naturalistic decision making for power system operators," *International Journal of Human-Computer Interaction*, 26(2-3), 278-291, 2010. doi:10.1080/10447310903499070

# How Things Can Fail



Real-world contextual elements and PIFs\* can include:

- Specific conditions (e.g., problematic components, mixed crews)
- Scenario dynamics (e.g., shift changes, multiple system shocks)
- Economic concerns
- Social behaviors and relationships

\*Usually referred to as “Performance Shaping Factors” (PSFs)

# Fundamental Probabilistic Model

- Human Error Probability (HEP)
  - Quantifies aleatory uncertainty
  - Is subject to epistemic uncertainties
  - Is a function of the task, the scenario context leading up to the task, and the relevant PIFs

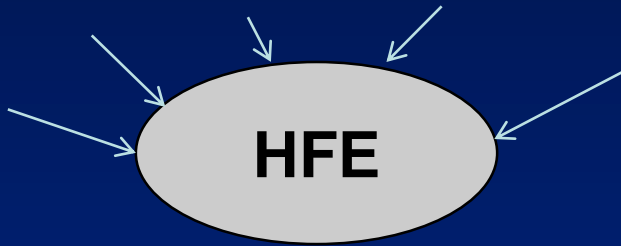
$$HEP \equiv P\{HFE\} = f(\text{task}, \text{scenario context}, \underline{PIF})$$

- Underlying assertion: human actions are predictable (in a probabilistic sense)
  - Performance of specific tasks, often with specific procedures and training
  - Bounded rationality: operators/staff are trying to do the right thing
- Note: HEP functional behavior on PIFs is usually assumed to be multiplicative, but other data might support additivity

# HRA Approaches

## Holistic Analysis (ATHEANA, MERMOS)

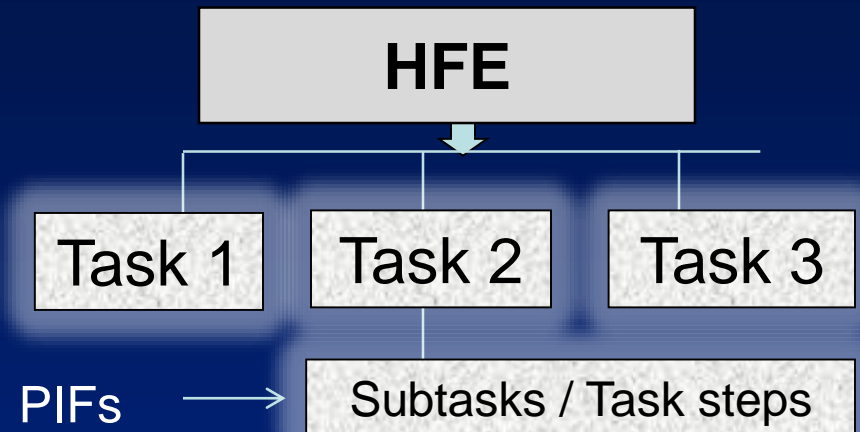
Tasks + Context (plant situation, scenario, and crew factors)



- Analyze context and develop operational story / narrative
- Identify situations deviating from the base story that lead to undesired actions
- Estimate the HEPs of the deviations

Strengths – Preserves context; uses expert ability to integrate complex information  
Limitations – Level of effort; subjectivity and variability

## Decomposition-Based Analysis (THERP, SPAR-H, CBDT, etc.)



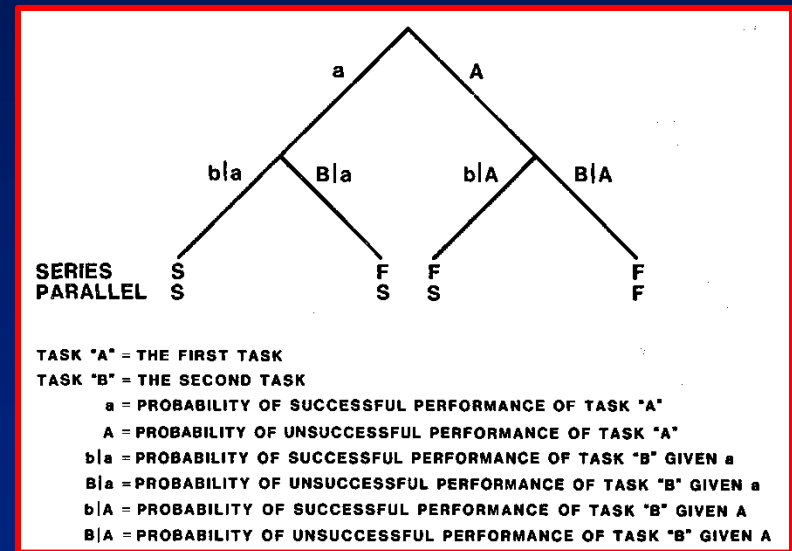
- Decompose HFE into tasks, possibly subtasks / steps
- Analyze PIFs for the lowest decomposition level
- Calculate HEP of every part, combine HEPs for the event

Strengths – Transparency; consistency  
Limitations – Formulaic; loss of context, interactions, non-linearities



# Technique for Human Error Rate Prediction (THERP)

- Widely-used HRA method, based on research started in 1976
- Task-oriented, focus on rule-based behavior (but also includes a time-reliability correlation for diagnosis)
- Task successes and failures represented with HRA event tree
- Tables used to quantify task success/failure probabilities
  - Some empirical basis
  - Considerable expert judgment
- Provides modifiers for dependent actions



NUREG/CR-1278

# Human Cognitive Reliability/Operator Reliability Experiment (HCR/ORE)

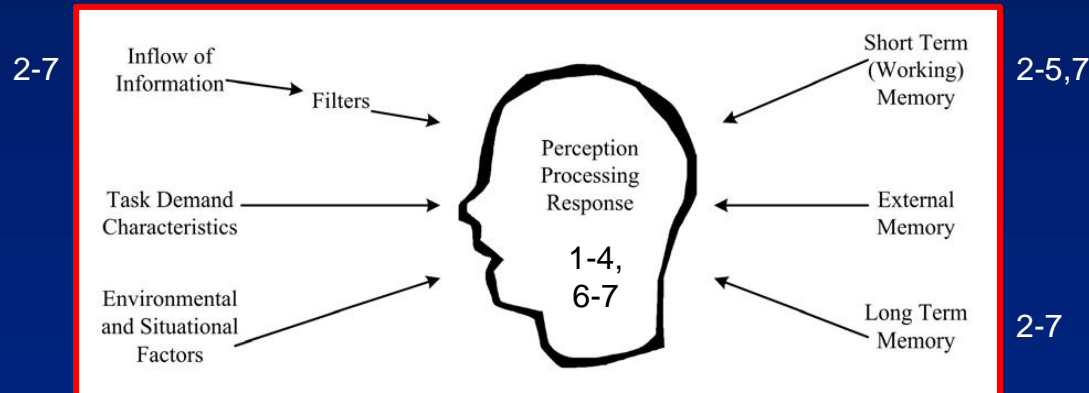
- Extension of HCR method (which was based on skill/rule/knowledge base categorization of actions)
- Focused on probability of non-response
  - Non-response = failure to diagnose OR failure to initiate response in a timely manner
  - Normalized correlations for groups of HFEs (“human interactions”) categorized by cue-response characteristics.
  - Analyst estimates median response time and time window; model provides non-response probability.
  - Has no “floor” for very large time margins
- Included in EPRI HRA Calculator

## Cause-Based Decision Tree (CBDT)

- Originally a supplement to HCR/ORE, now a standalone method in the EPRI HRA Calculator
- Eight decision trees used to develop non-response probabilities, considering multiple PIFs (e.g., training quality, procedures, human-machine interface)
  - 1) Relevant data not available
  - 2) Data not attended to
  - 3) Data errors
  - 4) Data misleading
  - 5) Procedure steps missed
  - 6) Misinterpretation of instructions
  - 7) Errors in interpreting logic
  - 8) Deliberate violations
- Initial non-response probabilities modified by a time-based recovery factor and added to the probabilities of execution failure

# Standardized Plant Analysis Risk – HRA (SPAR-H)

- Developed to support SPAR models, event and condition assessments
- Derived from THERP, multiple PIFs (PSFs) aggregated into eight groups based on information processing model



- |                            |   |
|----------------------------|---|
| 1) Available time          | 5) Procedures (including job aids)        |
| 2) Stress and stressors    | 6) Ergonomics and human-machine interface |
| 3) Complexity              | 7) Fitness for duty                       |
| 4) Experience and training | 8) Work processes                         |

# SPAR-H Worksheets

**A. Evaluate PSFs for the Diagnosis Portion of the Task, If Any.**

PSFs	PSF Levels	Multiplier for Diagnosis	Please note specific reasons for PSF level selection in this column.
Available Time	Inadequate time	P(failure) = 1.0 <input type="checkbox"/>	
	Barely adequate time ( $\approx 2/3$ x nominal)	10 <input type="checkbox"/>	
	Nominal time	1 <input type="checkbox"/>	
	Extra time (between 1 and 2 x nominal and > than 30 min)	0.1 <input type="checkbox"/>	
	Expansive time (> 2 x nominal and > 30 min)	0.01 <input type="checkbox"/>	
	Insufficient information	1 <input type="checkbox"/>	
Stress/Stressors	Extreme	5 <input type="checkbox"/>	
	High	2 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Complexity	Highly complex	5 <input type="checkbox"/>	
	Moderately complex	2 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Obvious diagnosis	0.1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Experience/Training	Low	10 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	High	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Procedures	Not available	50 <input type="checkbox"/>	
	Incomplete	20 <input type="checkbox"/>	
	Available, but poor	5 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Diagnostic/symptom oriented	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Ergonomics/HMI	Missing/Misleading	50 <input type="checkbox"/>	
	Poor	10 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Good	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Fitness for Duty	Unfit	P(failure) = 1.0 <input type="checkbox"/>	
	Degraded Fitness	5 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Work Processes	Poor	2 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Good	0.8 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	

Rev 1 (1/20/04)

**A. Evaluate PSFs for the Action Portion of the Task, If Any.**

PSFs	PSF Levels	Multiplier for Action	Please note specific reasons for PSF level selection in this column.
Available Time	Inadequate time	P(failure) = 1.0 <input type="checkbox"/>	
	Time available is $\approx$ the time required	10 <input type="checkbox"/>	
	Nominal time	1 <input type="checkbox"/>	
	Time available $\geq 5x$ the time required	0.1 <input type="checkbox"/>	
	Time available is $\geq 50x$ the time required	0.01 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Stress/Stressors	Extreme	5 <input type="checkbox"/>	
	High	2 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Complexity	Highly complex	5 <input type="checkbox"/>	
	Moderately complex	2 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Experience/Training	Low	3 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	High	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Procedures	Not available	50 <input type="checkbox"/>	
	Incomplete	20 <input type="checkbox"/>	
	Available, but poor	5 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Ergonomics/HMI	Missing/Misleading	50 <input type="checkbox"/>	
	Poor	10 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Good	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Fitness for Duty	Unfit	P(failure) = 1.0 <input type="checkbox"/>	
	Degraded Fitness	5 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	
Work Processes	Poor	5 <input type="checkbox"/>	
	Nominal	1 <input type="checkbox"/>	
	Good	0.5 <input type="checkbox"/>	
	Insufficient Information	1 <input type="checkbox"/>	

## **A Technique for Human Event Analysis (ATHEANA)**

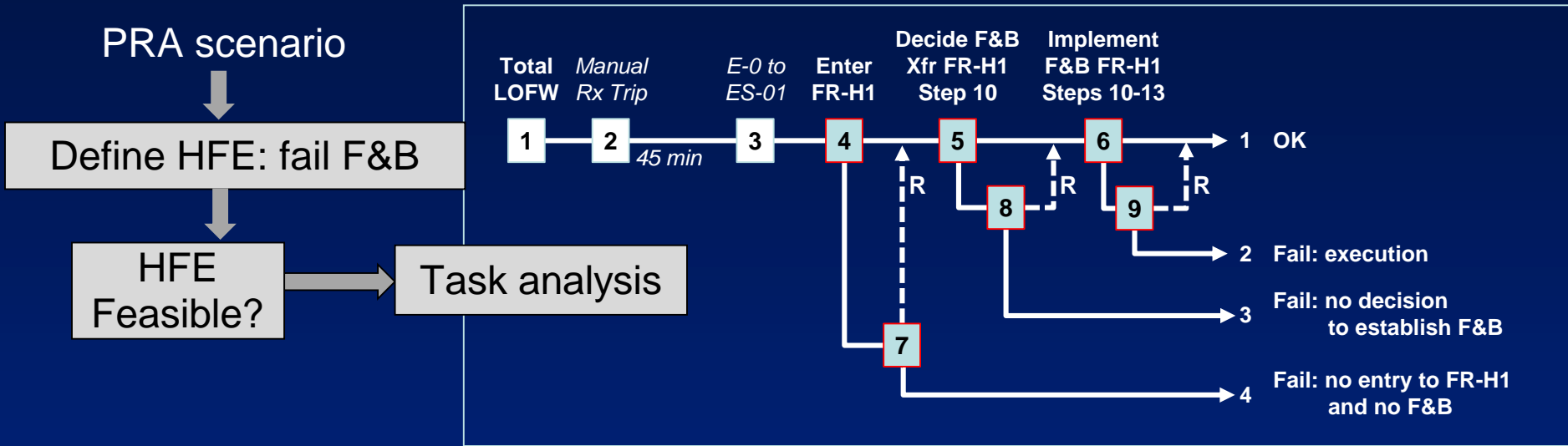
- Development started in support of low power and shutdown PRA (different conditions from at-power); evolved into general method
- Focuses on HFE context, identification of error-forcing conditions (EFCs)
- Does not use pre-established list of PIFs (PSFs)
- Holistic quantification via expert judgment; emphasizes involvement of knowledgeable plant staff (operations and training)

# Integrated Human Event Analysis System (IDHEAS)

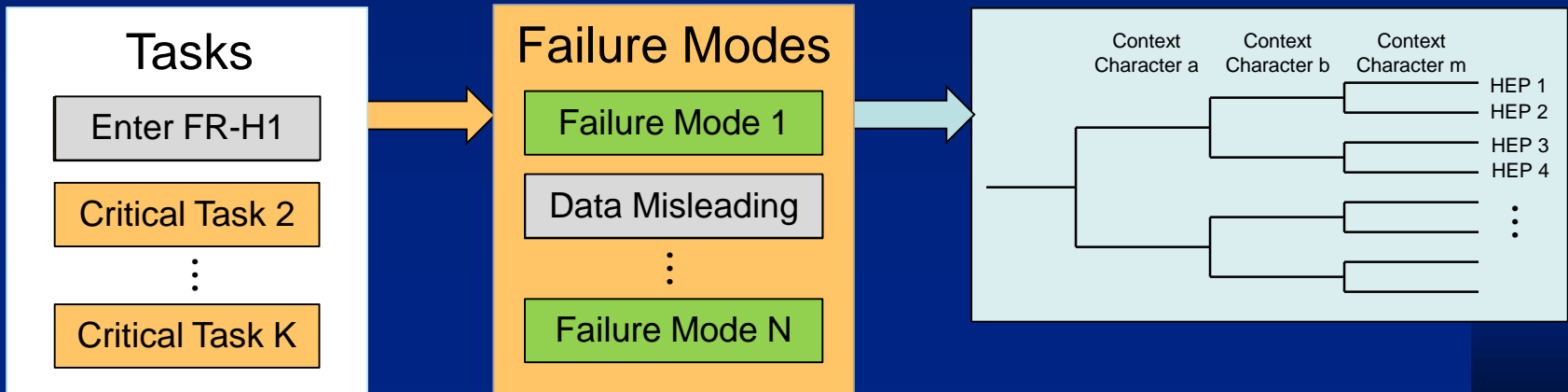
- Staff response to Commission direction “to evaluate the different human reliability models in an effort to propose a single model for the agency to use or guidance on which model(s) should be used in specific circumstances”
- General methodology + application modules
  - At-power
  - Event and condition assessment
- Decomposition-based, cognitive focus
- Supported by extensive review of human cognition literature (psychology, cognition, behavioral science, human factors) to identify relevant functions, mechanisms, and factors

# IDHEAS At-Power

## Qualitative analysis



## HEP quantification





# IDHEAS-G

Tasks are accomplished through the performance of various cognitive activities. These cognitive activities exercise general macrocognitive functions.

## Example

### Task:

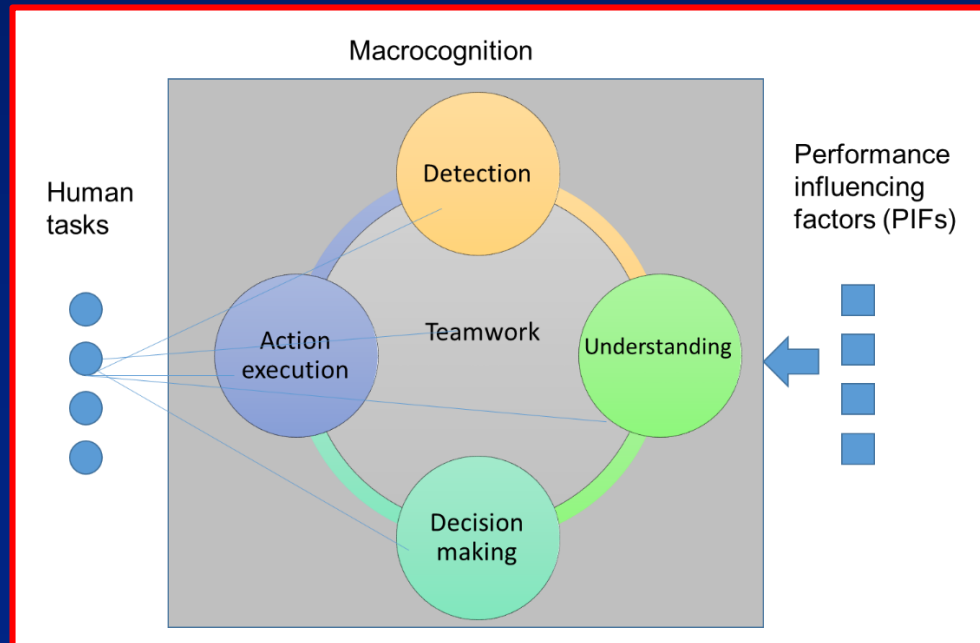
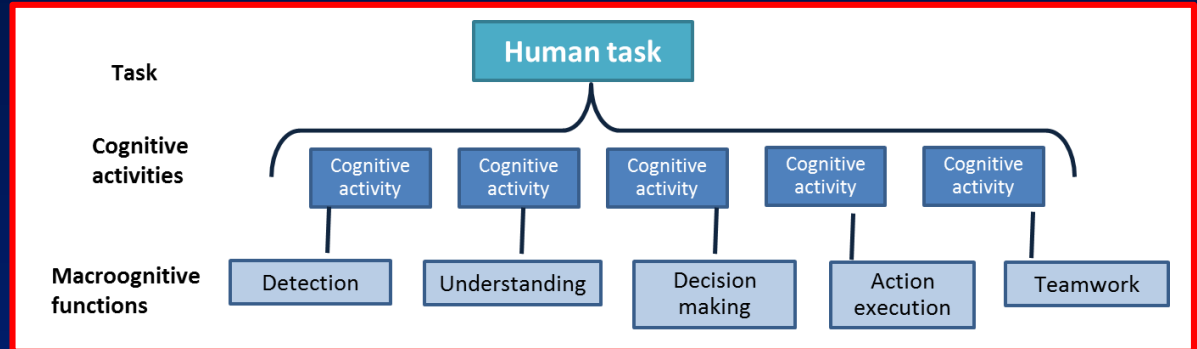
Identify Ruptured SG (as part of an action to isolate the ruptured SG)

### Cognitive Activities:

- Detect any one of:
  - unexpected rise in any SG NR level
  - high radiation level from any SG sample
  - high radiation from any SG steamline
  - high radiation from any SG blowdown
- Understand that any one signal provides indication of the faulted SG. Note:
  - The HRA-specified context includes successful reactor and turbine trip, energization of all AC buses, SI actuated, AFW available.
  - The specified context does not explicitly address the possibility of confounding signals and demands (e.g., alarms from unrelated SSCs not modeled in the PRA but demanding operator response.)

### Macrocognitive Functions:

- Detection
- Understanding



# IDHEAS-G

## Example

### Macroognitive Function:

- Detection

### Cognitive Process Elements:

- Establish mental model
- Select, identify, attend to information sources
- Perceive, recognize, classify information
- Verify, modify detection outcomes
- Retain, document/record, communicate outcomes

### Cognitive Mechanisms:

- Sensing
- Perception of sensing stimuli
- Vigilance maintenance
- ...

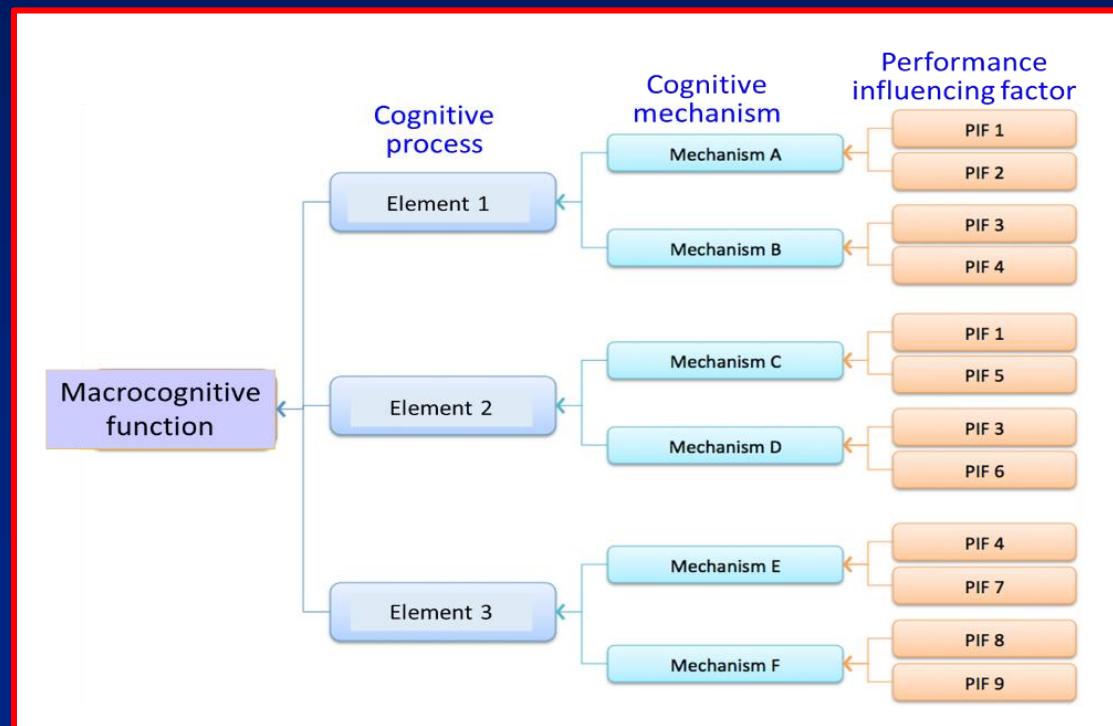
### “Capacity Limits”:

- Mismatch between sensory system and signal
- Weak signal
- Reduced vigilance due to sustained cognitive activities

### Performance Influencing Factors:

- Human-system interface
- Environmental factors
- Stress, time pressure, and anxiety
- Mental fatigue
- ...

Macroognitive functions are accomplished through a set of cognitive processes (“elements”) and cognitive processes are accomplished by cognitive mechanisms. Performance influencing factors affect how well the cognitive mechanisms are executed by challenging “capacity limits” for these mechanisms.



# IDHEAS-G

## Example

Task:  
Identify Ruptured SG (as part of an action to isolate the ruptured SG)

Macroognitive Function:

- Detection
- Understanding

Proximate Causes:

- Failure to perceive information
- Failure to attend to source of information
- ...

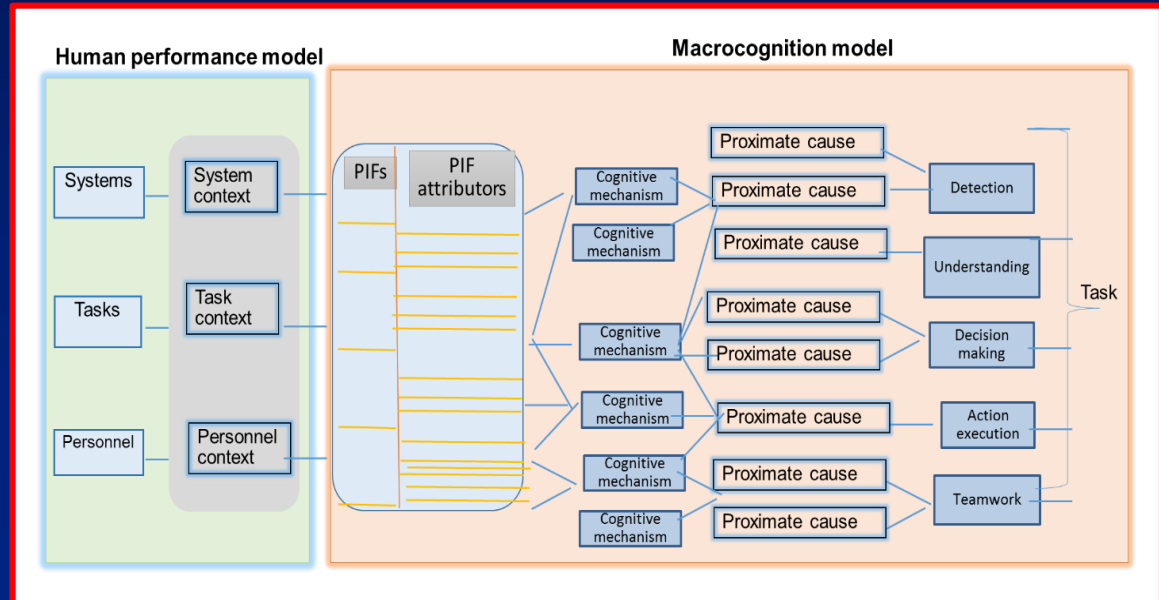
Cognitive Mechanisms:

- Sensing
- Perception of sensing stimuli
- Vigilance maintenance
- ...

Performance Influencing Factors:

- Human-system interface
- Environmental factors
- Stress, time pressure, and anxiety
- Mental fatigue
- ...

Task failure can be caused by failure of any single cognitive mechanism (which propagates through the cognitive process/macroognitive function/cognitive activity causality chain).\* Each potential failure of a cognitive process is a potential “proximate cause” for macroognitive function failure.



\*Note: from a systems point of view, a task is modeled as a series system with a very large number of potential single-point failures.

## HRA Guidance

- Many methods and viewpoints, but general agreement on high-level model and “good practices”
- NUREG-1792: high-level guidance, e.g.,
  - Perform field observations and discussions
  - Use screening values during initial quantification
  - Account for dependencies among HEPs
  - Evaluate the reasonableness of the HEPs
- NUREG-1842: evaluation of several methods against these good practices
- Various documents for specific applications, e.g., NUREG-1921 (fire HRA) and NUREG-1921 Supplement 1 (fire HRA, main control room abandonment)

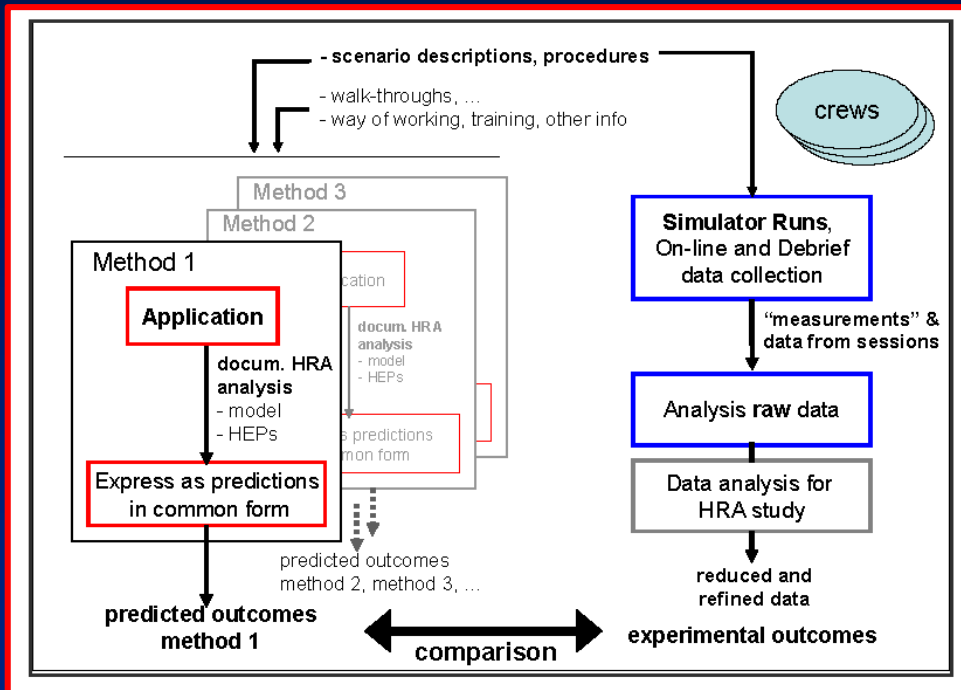
## Ispra Benchmark Exercises (1986-1988)

- European Commission Joint Research Centre
- Comparison of methods and modeling
- 15 teams, multiple methods *Validation*
- Test and maintenance
  - Failure to detect check valve failure, failure to restore system
  - Good agreement on qualitative characterization (key human error interactions and failure mechanisms), divergence on modeling and quantification
  - Some variance reduction when using a common model
- Complicated transient
  - LOOP, 2/4 EDGs fail to start, partial CCF of EFW valves
  - Differences in modeling (scope of analysis, aggregation) and quantification
  - Large method-to-method and team-to-team differences

# International HRA Empirical Studies

- OECD/NEA Halden Reactor Project
- Comparisons of analysis results with data from HAMMLAB simulator to identify strengths and weaknesses
- 14 operator crews, 13 HRA teams, “blind” study
- Operational transients:
  - Steam generator tube rupture (SGTR), loss of feedwater (LOFW)
  - Base case and complex, multiple HFEs with varying difficulty
- Findings include:
  - Large variations in how crews followed procedures
  - Large variations in HEPs; many rankings don’t reflect difficulty
  - Some analyses don’t strongly differentiate across HFEs
  - Methods that emphasize mechanisms and contextual factors provide richer (and often predictive) narratives, but not necessarily better HEPs

# Study Process



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## Challenges Include:

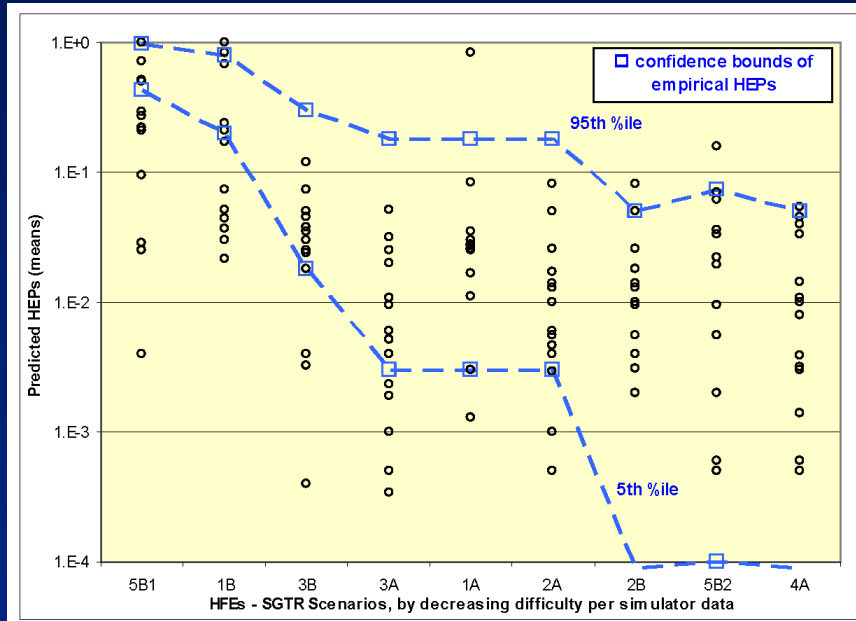
- Differences between HAMMLAB simulator and home plant
- Characterizing crew behaviors (e.g., drivers for performance)
- Statistically small sample
- Defining "failure" for intermediate HFEs

# US HRA Empirical Studies

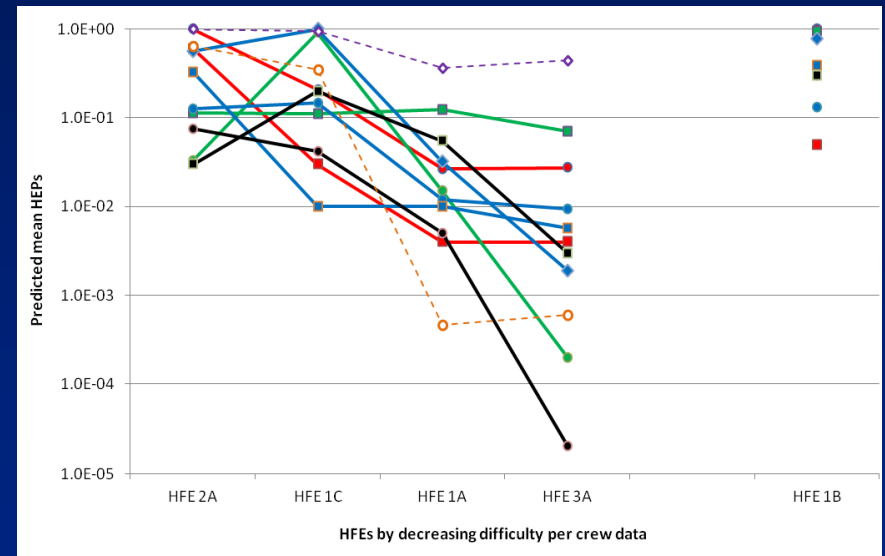
- Similar to international study but using a US PWR (simulator and crews). Also addressed concerns regarding
  - Lack of testing of team-to-team variability in using the same method
  - Inability of analysis teams to visit simulator, interview crews
- 4 crews, 9 HRA teams
- Operational transients:
  - LOFW followed by SGTR
  - Loss of component cooling water and RCP seal water
  - SGTR
- Findings include:
  - Less variability vs. HAMMLAB study and Ispra: HRA team learning? Better practiced with US crews? Plant visit?
  - Qualitative analyses can be improved
  - HRA improvements should focus on aiding analysts finding and characterizing contextual factors and mechanisms causing cognitive failures



# Comparing Predictions with Performance



NUREG-2127



NUREG-2156

# Technical Challenges

- Complicating factors
  - Specific conditions (e.g., pre-accident conditions including problematic components; specific crew on shift including makeup crews)
  - Scenario dynamics (e.g., mindset established by specific evolution, shift changes, multiple system shocks, changes in local environment, external directions)
  - Additional crew concerns (e.g., economic impact of action, offsite environment)
  - Social behaviors and relationships (e.g., trust within crew, between organizational elements, group behavior)

## Technical Challenges (cont.)

- Data from actual incidents
  - Statistically sparse, arguably unique characteristics for each event
  - Extremely rich qualitative information for a few events
- Data from other simulator exercises: transferability to HRA/PRA
  - Design and operational differences
  - Data collection protocols
- Technology advances affecting human performance
  - Advanced control rooms
  - Smart/distributed technology
  - Remote operations
  - ...

## Socio-Organizational Challenges

- Multiple technical disciplines with varying goals, views on the meaningfulness of a PRA-oriented HRA, views on needed rigor
- Interdisciplinary trust
  - HRA developers: academic/professional reward system => proliferation of HRA methods
  - PRA analysts: need for “now” answers => development of “good enough” methods, resistance to change
  - PRA users: discomfort with large uncertainties => dismissal/discounting of results and insights
  - Science critics: weaknesses in current methods/models => “house of cards” view on PRA and RIDM affecting willingness to help

# Grand Challenge – Incorporating Organizational Factors

- Long-recognized as an important influence
  - Culture and climate
  - Resources
  - Direct involvement in events
- Scope >> current PRA scope
  - Time
  - Organizations (functions and structure)
  - Space
  - Technical disciplines
- Data
  - Availability
  - Quality
- Non-monotonic effects

## Non-Monotonic Effects: Examples

- Good safety culture can reduce worker risk but increase plant-level risk
  - Pre-emptive reactor trip on loss of communications with diver
  - Reluctance to send workers to hazardous areas
- “Forceful leadership” can overcome organizational inertia but can also stifle important views