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July 28, 1976

Director of Nuclear Reactor Regulation Att: Mr Albert Schwencer, Chief Operating Reactor Branch No 1 US Nuclear Regulatory Commission Washington, DC 20555

DOCKET 50-255, LICENSE DPR-20
PALISADES PLANT, VESSEL TRANSIENT LOADS

Your letter dated June 7, 1976 requested that we inform you within 30 days of our schedule for providing you with our evaluation of the adequacy of the Palisades Plant pressure vessel supports.

Consumers Power has been studying vessel transient load problems for a number of months and has also participated in a Combustion Engineering User Group involved in studying this problem. Part of this effort has involved a probability study of a commercial reactor coolant system similar to our Palisades Plant. This study performed by Science Applications, Incorporated (SAI), was made to determine the probability of a failure of the reactor coolant system in the annular region between the reactor vessel and cavity shield wall. The results of this study were presented to members of your staff on July 13, 1976 and will be submitted formally on about August 16, 1976.

We have concluded that the probability of a failure of the reactor coolant system in the annular region between the reactor vessel and the cavity shield wall is acceptably small. It is our opinion, based on this low probability and the design bases of our Palisades Plant, that further reactor vessel support reassessment would fall under the backfitting rule of 10 CFR 50.109 and in any event does not need to be considered further.

Following your review of this study, we would be glad to meet with you to discuss this issue further and determine the course of action necessary to address any further concerns you might have.

David A. Bixel

Assistant Nuclear Licensing Administrator

David a Bifel/NB

CC: JGKeppler, USNRC

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