

**Session: Inspection Oversight** 

Inspection Oversight and Regulatory Experience Related to the Review of 10 CFR 72.48 Evaluations

> Division of Spent Fuel Management Regulatory Conference 2018 December 11 - 12, 2018

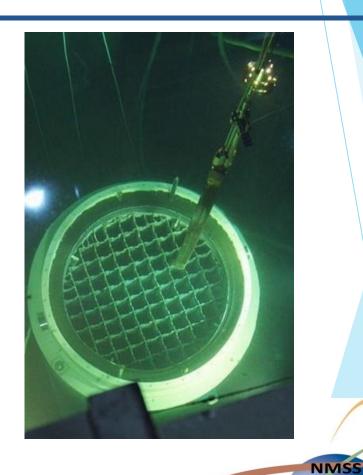
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## **Content**

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- Overview of Regulatory Experience and Practices
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#### Introduction

 The purpose of this presentation is to provide a brief overview of the U.S. Nuclear Regulatory Commission (NRC) regulatory experience related to the inspection oversight of the review of 10 CFR 72.48 evaluations and provide information on the endorsement of the new guidance document.



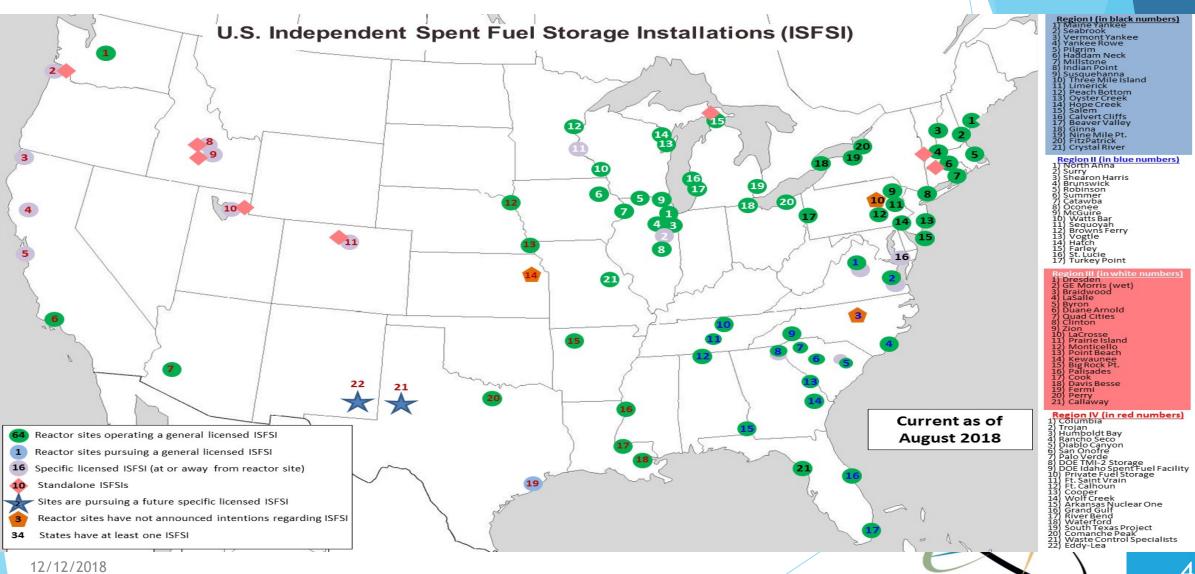
 Success for this presentation is to ensure that all interested parties have a general awareness of recent NRC regulatory experience and provide the status on the new guidance document.





Protecting People and the Environment

## **U.S. NRC Regulatory Experience**





## Certificate Holders: with NRC-Approved Designs for Use



NAC International



Holtec International



Orano Transnuclear (TN) Americas



Energy Solutions





#### **Dry Cask Storage Safety Inspections**

# NRC Inspection Manual Chapters (IMC)s

- IMC 2690 Inspection Program Guidance for Dry Storage of Spent Reactor Fuel at Independent Spent Fuel Storage Installations (ISFSI)s
- IMC 2515, Appendix C Special and Infrequently Performed Inspections

## NRC Inspection Procedure (IP)

- IP 60851 Design Control of ISFSI Components
- IP 60853 Onsite Fabrication of Components and Construction of an ISFSI
- IP 60854 Preoperational Testing of an ISFSI
- IP 60855 Operation of an ISFSI
- IP 60856 Review of 10 CFR 72.212(b) Evaluations
- IP 60857 Review of 10 CFR 72.48 Evaluations





#### IP 60857 Inspection Requirements

# Inspection Sample Selection

- Review a list of evaluations performed and select a representative sample based on a judgment of risk significance
- Select a sample that were screened out as not requiring a performance of the 10 CFR 72.48 evaluation

## Review of Evaluations

- Review the selected sample evaluations to verify that the licensee or certificate holder has appropriately considered the conditions under which the change was made
- Review the selected screen to determine that an evaluation was not required and verify that the conclusions were correct and consistent with the licensee's 10 CFR 72.48 procedure

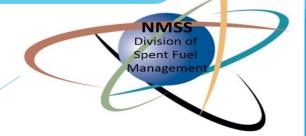




#### Inspection Guidance

Regulatory Guide 3.72, "Guidance for Implementation of 10 CFR 72.48, Changes, Test, and Experiments," Revision 0

Nuclear Energy Institute (NEI) 96-07, Appendix B dated March 5, 2001





# Overview - Applicability Process

# Applicability Determination

- Applicability Determination is to determine the correct regulatory change control process, if any for all or part of a proposed activity
- Generally three to four questions to determine if the proposed activity is applicable





## **Overview - Screening Process**

Screening for Adverse Effects

- Screening is the process for determining whether a proposed activity requires a 10 CFR 72.48 evaluation to be performed
- Generally three to four questions to determine if the proposed activity is adversely impacted

Screening for Methods of Evaluation

- Determine if an activity involves a method of evaluation (MOE)
- Distinguishing between input parameters and elements





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# Overview - 72.48(c)(2) Criteria - Evaluation

10 CFR 72.48 Criterion (c)(2)i  Does the proposed activity result in more than a minimal increase in the frequency of occurrence of an accident previously evaluated in the FSAR (as updated)?

10 CFR 72.48 Criterion (c)(2)ii  Does the proposed activity result in more than a minimal increase in the likelihood of occurrence of a malfunction of a system, structure, or component (SSC) important to safety previously evaluated in the FSAR (as updated)?



# Overview - 72.48(c)(2) Criteria - Evaluation

10 CFR 72.48 Criterion (c)(2)iii  Does the proposed activity result in more than a minimal increase in the consequences of an accident previously evaluated in the FSAR (as updated)?

10 CFR 72.48 Criterion (c)(2)iv

• Does the proposed activity result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the FSAR?

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## Overview - 72.48(c)(2) Criteria - Evaluation

10 CFR 72.48 Criterion (c)(2)v

• Does the proposed activity create a possibility for an accident of a different type than any previously evaluated in the FSAR (as updated)?

10 CFR 72.48 Criterion (c)(2)vi • Does the proposed activity create a possibility for a malfunction of an SSC important to safety with a different result than any previously evaluated in the FSAR (as updated)?





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# Overview - 72.48(c)(2) Criteria - Evaluation

10 CFR 72.48 Criterion (c)(2)vii  Does the proposed activity result in a design basis limit for a fission product barrier as described in the FSAR (as updated) being exceeded or altered?

10 CFR 72.48 Criterion (c)(2)viii  Does the proposed activity result in a departure from a method of evaluation described in the FSAR (as updated) used in establishing the design bases or in the safety analyses?





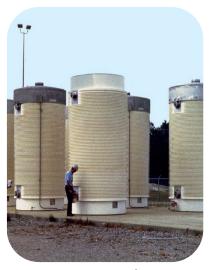
## Regulatory Experience



Method of Evaluation (MOE)



Screenings vs. Evaluations



Meeting the applicable criteria of 10 CFR 72.48(c)(2)



Nonconforming or Degraded Conditions





## Overview

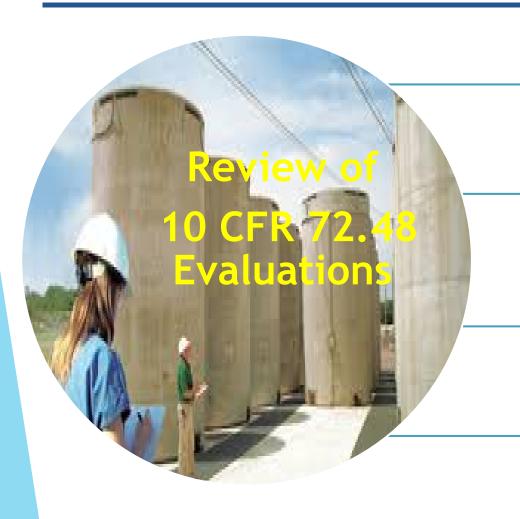
## Regulatory Guide 3.72 Update

- The staff developed a Draft Regulatory Guide (DG) for a revision to Regulatory Guide 3.72," Guidance For Implementation of 10 CFR 72.48, Changes, Test, and Experiments"
- The staff expects the DG to be out for public comment within the next six months.





## Summary





Method of Evaluation



Meeting the applicable criteria of 10 CFR 72.48(c)(2)



Screenings vs. Evaluations

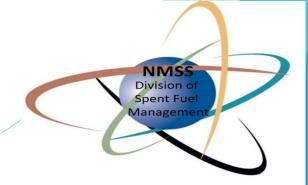


Nonconforming or Degraded Conditions











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### References

- Code of Federal Regulations (CFR), Title 10, Part 72 Licensing Requirements for the Independent Storage of Spent Nuclear Fuel, High-Level Radioactive Waste, and Reactor-Related Greater Than Class C Waste.
- Inspection Manual Chapter (IMC) 2690, Inspection Program for Dry Storage of Spent Reactor Fuel at Independent Spent Fuel Storage Installations and for 10 CFR Part 71 Transportation Packaging
- Regulatory Guide 3.72, Guidance for Implementation of 10 CFR 72.48, Changes, Tests, and Experiments