



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION I  
2100 RENAISSANCE BOULEVARD, SUITE 100  
KING OF PRUSSIA, PA 19406-2713

November 8, 2018

Mr. Bryan C. Hanson  
Senior Vice President, Exelon Generation Company, LLC  
President and Chief Nuclear Officer, Exelon Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – INTEGRATED  
INSPECTION REPORT 05000333/2018003

Dear Mr. Hanson:

On September 30, 2018, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at the James A. FitzPatrick Nuclear Power Plant. On October 22, 2018, the NRC inspectors discussed the results of this inspection with Mr. Joseph Pacher, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

The NRC inspectors did not identify any findings or violations of more than minor significance.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR), Part 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

*/RA/*

Anthony Dimitriadis, Chief  
Reactor Projects Branch 5  
Division of Reactor Projects

Docket Number: 50-333  
License Number: DPR-59

Enclosure:  
Inspection Report 05000333/2018003

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INSPECTION REPORT 05000333/2018003 DATED NOVEMBER 8, 2018

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**U.S. NUCLEAR REGULATORY COMMISSION  
Inspection Report**

Docket Number: 50-333

License Number: DPR-59

Report Number: 05000333/2018003

Enterprise Identifier: I-2018-003-0065

Licensee: Exelon Generation Company, LLC (Exelon)

Facility: James A. FitzPatrick Nuclear Power Plant

Location: Scriba, NY

Inspection Dates: July 1, 2018 to September 30, 2018

Inspectors: J. Schussler, Acting Senior Resident Inspector  
G. Stock, Acting Senior Resident Inspector  
J. Bream, Physical Security Inspector  
P. Boguszewski, Project Engineer  
T. Daun, Resident Inspector, Susquehanna Nuclear Power Plant  
J. DeBoer, Emergency Preparedness Inspector  
S. Ghayeb, Acting Resident Inspector  
E. Miller, Senior Resident Inspector, Nine Mile Point Nuclear Station  
M. Modes, Senior Reactor Inspector  
R. Rolph, Health Physicist  
B. Sienel, Resident Inspector, Nine Mile Point Nuclear Station

Approved By: Anthony Dimitriadis, Chief  
Reactor Projects Branch 5  
Division of Reactor Projects

Enclosure

## **SUMMARY**

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring Exelon's performance at FitzPatrick by conducting the baseline inspections described in this report in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

No findings or more-than-minor violations were identified.

## PLANT STATUS

FitzPatrick began the inspection period at approximately 98 percent and gradually lowered power as it entered end-of-cycle coastdown operations. On September 10, 2018, operators commenced a shutdown from approximately 80 percent for a planned refueling and maintenance outage (RFO23). The unit remained shutdown for the remainder of the inspection period.

## INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed plant status activities described in IMC 2515, Appendix D, "Plant Status," and conducted routine reviews using IP 71152, "Problem Identification and Resolution." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess Exelon's performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

## REACTOR SAFETY

### 71111.04 - Equipment Alignment

#### Partial Walkdown (4 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) 'A' and 'B' emergency service water on July 9, 2018
- (2) Reactor core isolation cooling (RCIC) system on July 12, 2018
- (3) 'B' and 'D' emergency diesel generators (EDGs) on July 25, 2018
- (4) Decay heat removal system on September 27, 2018

#### Complete Walkdown (1 Sample)

The inspectors evaluated system configurations during a complete walkdown of the 'B' residual heat removal system on September 12, 2018.

### 71111.05A/Q - Fire Protection Annual/Quarterly

#### Quarterly Inspection (8 Samples)

The inspectors evaluated fire protection program implementation in the following selected areas:

- (1) Screenwell, fire area (FA) SH-1, on July 5, 2018
- (2) Safety pump rooms, FA SP-1, FA SP-2, FA FP-1, and FA-FP-3, on July 5, 2018

- (3) Auxiliary boiler room, FA AS-1, on July 9, 2018
- (4) Cable spreading room, FA CS-1, on July 9, 2018
- (5) Standby gas treatment room, FA CS-1, on July 9, 2018
- (6) Turbine deck, condenser bay, and feedwater heater bays, FA TB-1, on September 13, 2018
- (7) Outboard main steam isolation valve (MSIV) room and steam tunnel, FA TB-1, on September 20, 2018
- (8) Drywell, FA PC-1, on September 25, 2018

#### 71111.07 - Heat Sink Performance

##### Heat Sink (1 Sample)

The inspectors evaluated Exelon's monitoring and maintenance of the 'A' residual heat removal heat exchanger performance.

#### 71111.08 – Inservice Inspection Activities (1 Sample)

The inspectors evaluated Exelon's non-destructive examination and welding activities at FitzPatrick by reviewing the volumetric examination by manual ultrasonic encoded examination of Reactor Nozzle N1 and Reactor Nozzle N2 during the period September 17- 20, 2018, for compliance to the requirements in Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code.

#### 71111.11 - Licensed Operator Requalification Program and Licensed Operator Performance

##### Operator Requalification (1 Sample)

The inspectors observed and evaluated a simulator scenario including a trip of the 'A' core spray loop holding pump, a trip of the 'A' condensate pump, a trip of the 'A' service water pump and failure of the standby pump to auto-start, a loss of control rod drive, and an unisolable high pressure coolant injection (HPCI) steam leak into the reactor building requiring emergency depressurization on July 17, 2018.

##### Operator Performance (1 Sample)

The inspectors observed operations personnel during downpower to 15 percent after end-of-cycle coastdown to open the unit output breakers for the start of RFO23 on September 9, 2018.

#### 71111.12 - Maintenance Effectiveness

##### Routine Maintenance Effectiveness (1 Sample)

The inspectors evaluated the effectiveness of routine maintenance activities associated with the following equipment and/or safety significant functions:

- (1) Flexible mitigating strategies equipment on August 6, 2018

#### 71111.13 - Maintenance Risk Assessments and Emergent Work Control (6 Samples)

The inspectors evaluated the risk assessments for the following planned and emergent work activities:

- (1) 'B' service air compressor out of service for maintenance on July 31, 2018
- (2) 'B' core spray system out of service for maintenance on August 1, 2018
- (3) 'A' core spray system inoperable for hold pump planned maintenance on August 9, 2018
- (4) Differential voltage relay testing on both offsite power lines on August 31, 2018
- (5) Phase 1 outage risk management on September 12, 2018
- (6) Station battery 'A' out of service for maintenance on September 20, 2018

#### 71111.15 - Operability Determinations and Functionality Assessments (3 Samples)

The inspectors evaluated the following operability determinations and functionality assessments:

- (1) RCIC operability with condensate storage tank (CST) low-level transmitter 13LS-76B inoperable on August 16, 2018
- (2) Pitting identified on drywell liner on September 24, 2018
- (3) 'A' and 'C' residual heat removal service water pump bolt material type on September 30, 2018

#### 71111.18 - Plant Modifications (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) Engineering Change 620602 – Replace all 11 two-stage safety relief valves with three-stage valves, on September 30, 2018

#### 71111.20 - Refueling and Other Outage Activities

The inspectors evaluated RFO23 activities from September 10, 2018 through September 30, 2018.

#### 71111.19 - Post Maintenance Testing (7 Samples)

The inspectors evaluated post maintenance testing for the following maintenance/repair activities:

- (1) Torus leak rate outboard isolation valve testing following unplanned maintenance on July 17, 2018
- (2) Diesel fire pump 76-P1 relief valve replacement on July 19, 2018
- (3) Torus leak rate outboard isolation valve maintenance and testing following unplanned maintenance on August 26, 2018
- (4) RCIC transfer switch from CST suction to Torus suction following unplanned maintenance on September 4, 2018
- (5) Low pressure coolant injection inverter replacement and testing on September 26, 2018
- (6) MSIV 85 percent open position switch replacement on September 28, 2018
- (7) Residual heat removal 'A' containment spray outboard isolation valve 10MOV-26A following planned maintenance on September 30, 2018

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Routine (3 Samples)

- (1) ST-9BA, 'A' and 'C' EDG Full Load Test and Emergency Service Water Pump Operability Test, on July 9, 2018
- (2) ST-1B, MSIV Fast Closure Test, on September 12, 2018
- (3) ST-9CA, 'A' Division Loss of Offsite Power/Loss of Cooling Accident Test, on September 29, 2018

Inservice (1 Sample)

- (1) ST-6HB, 'B' Standby Liquid Control Quarterly Operability Test, on July 26, 2018

Containment Isolation Valve (1 Sample)

- (1) ST-39B-X7A, 'A' MSIV As-Found Local Leak Rate Test, on September 14, 2018

71114.02 - Alert and Notification System Testing (1 Sample)

The Inspectors evaluated the maintenance and testing of the alert and notification system (ANS) from July 2016 to July 2018.

71114.03 - Emergency Response Organization Staffing and Augmentation System (1 Sample)

The inspectors conducted a review of Exelon's emergency response organization (ERO) augmentation staffing requirements and the process for notifying and augmenting the ERO.

71114.05 - Maintenance of Emergency Preparedness (1 Sample)

The inspectors reviewed a number of activities to evaluate the efficacy of Exelon's efforts to maintain FitzPatrick's emergency preparedness programs.

71114.06 - Drill EvaluationEmergency Planning Drill (1 Sample)

The inspectors evaluated the conduct of a routine emergency planning drill observation on August 1, 2018.

Drill/Training Evolution (1 Sample)

The inspectors evaluated a simulator scenario including a trip of the 'A' core spray loop holding pump, a trip of the 'A' condensate pump, a trip of the 'A' service water pump and failure of the standby pump to auto-start, a loss of control rod drive, and an un-isolable HPCI steam leak into the reactor building which required a site area emergency declaration on July 17, 2018.



## **RADIATION SAFETY**

### 71124.01 - Radiological Hazard Assessment and Exposure Controls

#### Radiological Hazard Assessment (1 Sample)

The inspectors reviewed recent plant radiation surveys to ensure radiological hazards to onsite workers and to members of the public were identified. The inspectors evaluated assessments and controls for new radiological hazards.

#### Instructions to Workers (1 Sample)

The inspectors evaluated instructions provided to workers.

#### Contamination and Radioactive Material Control (1 Sample)

The inspectors observed the monitoring of potentially contaminated material leaving the radiological control area and evaluated contamination and radioactive material controls. The inspectors selected several sealed sources from inventory records and assessed whether the sources were accounted for and tested for loose surface contamination.

#### Radiological Hazards Control and Work Coverage (1 Sample)

The inspectors evaluated in-plant radiological conditions and performed independent radiation measurements during facility walk-downs and observation of radiological work activities. The inspectors assessed posted surveys, radiation work permits, worker radiological briefings, the use of continuous air monitoring, and dosimetry monitoring to determine if they were consistent with the present conditions.

#### High Radiation Area and Very High Radiation Area Controls (1 Sample)

The inspectors evaluated risk-significant high radiation area and very high radiation area controls including postings and physical barriers.

#### Radiation Worker Performance and Radiation Protection Technician Proficiency (1 Sample)

The inspectors evaluated radiation worker and radiation protection technician radiological performance.

### 71124.02 - Occupational As Low As Reasonably Achievable (ALARA) Planning and Controls

#### Verification of Dose Estimates and Exposure Tracking Systems (1 Sample)

The inspectors evaluated dose estimates and exposure tracking.

#### Radiation Worker Performance (1 Sample)

The inspectors evaluated radiation worker and radiation protection technician performance.

### 71124.03 - In-Plant Airborne Radioactivity Control and Mitigation

#### Engineering Controls (1 Sample)

The inspectors observed the setup and use of ventilation systems and the use of monitoring instrumentation.

### 71124.04 - Occupational Dose Assessment

#### Source Term Characterization (1 Sample)

The inspectors evaluated Exelon's source term characterization at FitzPatrick.

#### Special Dosimetric Situations (1 Sample)

The inspectors evaluated Exelon's performance for special dosimetric situations.

### 71124.05 - Radiation Monitoring Instrumentation

#### Walkdowns and Observations (1 Sample)

The inspectors evaluated radiation monitoring instrumentation during plant walkdowns.

#### Calibration and Testing Program (1 Sample)

The inspectors evaluated Exelon's calibration and testing program.

### 71124.06 - Radioactive Gaseous and Liquid Effluent Treatment

#### Walkdowns and Observations (1 Sample)

The inspectors walked down the gaseous and liquid radioactive effluent monitoring and filtered ventilation systems to assess the material condition and verify proper alignment according to plant design.

#### Calibration and Testing Program (Process and Effluent Monitors) (1 Sample)

The inspectors evaluated Exelon's gaseous and liquid effluent monitor instrument calibration and testing.

#### Sampling and Analyses (1 Sample)

The inspectors evaluated radioactive effluent sampling and analysis activities.

#### Instrumentation and Equipment (1 Sample)

The inspectors reviewed radioactive effluent discharge system surveillance test results and reviewed the methodology used to determine the radioactive effluent stack and vent flow rates based on technical specifications/off site dose calculation manual (TS/ODCM) acceptance criteria.

### Dose Calculations (1 Sample)

The inspectors reviewed several liquid and gaseous discharges to evaluate public dose calculations (monthly, quarterly, and annual) and the annual radiological effluent release reports for 2017 and 2018.

## **OTHER ACTIVITIES – BASELINE**

### 71151 - Performance Indicator Verification (12 Samples)

The inspectors verified Exelon performance indicators submittals listed below for the period from December 2017 through June 2018:

- (1) EP01, Drill and Exercise Performance
- (2) EP02, ERO Drill Participation
- (3) EP03, ANS System Reliability
- (4) IE01, Unplanned Scrams
- (5) IE04, Unplanned Scrams with Complications
- (6) MS06, MSPI Emergency AC Power Systems
- (7) MS07, MSPI High Pressure Injection
- (8) MS08, MSPI Heat Removal Systems
- (9) MS09, MSPI Residual Heat Removal Systems
- (10) MS10, MSPI Cooling Water Systems

The inspectors verified Exelon performance indicator submittals listed below for the period from January 2017 through June 2018:

- (11) PR01, RETS/ODCM Radiological Effluent Occurrence

The inspectors verified Exelon performance indicator submittals listed below for the period from August 2017 through August 2018:

- (12) OR01, Occupational Exposure Control

## **EXIT MEETINGS AND DEBRIEFS**

The inspectors verified no proprietary information was retained or documented in this report.

- On July 19, 2018, the inspectors conducted a debrief meeting and presented the baseline radiation protection inspection results to Mr. Chris Adner, Acting Plant Manager, and other members of the Exelon staff.
- On July 19, 2018, the inspectors conducted a debrief meeting and presented the preliminary emergency preparedness inspection results to Mr. Chris Adner, Acting Plant Manager, and other members of the Exelon staff.
- On August 16, 2018, the inspectors conducted a debrief meeting and presented the baseline radiation protection inspection results to Mr. Timothy Peters, Plant Manager, and other members of the Exelon staff.

- On September 27, 2018, the inspectors conducted a debrief meeting and presented the radiation protection inspection results to Mr. Joseph Pacher, Site Vice President, and other members of the Exelon staff.
- On October 22, 2018, the inspectors presented the quarterly resident inspection results to Mr. Joseph Pacher, Site Vice President, and other members of the Exelon staff.

**DOCUMENTS REVIEWED****71111.04**Procedures

AOP-68A, Loss of Decay Heat Removal, Revision 7  
 FM-133A, Decay Heat Removal Flow Diagram, Revision 9  
 FM-133B, Decay Heat Removal Flow Diagram, Revision 8  
 OP-13D, Residual Heat Removal Shutdown Cooling, Revision 31  
 OP-21, Emergency Service Water, Revision 38  
 OP-22, Diesel Generator Emergency Power, Revision 62  
 OP-30B, Decay Heat Removal System, Revision 19  
 OP-60, Diesel Generator Room Ventilation, Revision 9

Issue Reports

04092020	04099657	04117175	04117285
04117659			

Drawings

FM-20A, Flow Diagram Residual Heat Removal System 10, Revision 072  
 FM-20B, Flow Diagram Residual Heat Removal System 10, Revision 073  
 FM-22A, Flow Diagram Reactor Core Isolation Cooling System 13, Revision 57  
 FM-46B, Flow Diagram Emergency Service Water System 46 and 15, Revision 57  
 ISI-FM-20A, Flow Diagram Residual Heat Removal System 10, Revision 18  
 ISI-FM-20B, Flow Diagram Residual Heat Removal System 10, Revision 16

Miscellaneous

Engineering Report No. JAF-RPT-05-AMM02, Aging Management Review of Residual Heat Removal System, Revision 3  
 FPSSK-100, Fire Barrier Penetrations Arrangement Plan Below Elevation 272' 0", Revision 5  
 FPSSK-206, Fire Barrier Penetrations Arrangement Fire Area Zone XII/SP-1 Elevation 255' 0", Revision 2

**71111.05**Procedures

PFP-PWR09, Auxiliary Boiler Room, Elevation 272' and 285', Revision 4  
 PFP-PWR11, Cable Spreading Room, Elevation 272', Revision 3  
 PFP-PWR18, Drywell Area Elevation 268', Fire Area Zone XIV/PC-1, Revision 0  
 PFP-PWR19, Drywell Area Elevation 292', Fire Area Zone XIV/PC-1, Revision 0  
 PFP-PWR22, Standby Gas Filter Room. Elevation 272', Revision 6  
 PFP-PWR34, Screenwell House and Water Treatment Area, Elevation 235', 255', and 260', Revision 05  
 PFP-PWR33, Pump Rooms (Screenwell), Elevation 255', Revision 02  
 PFP-PWR42, Turbine Building - North / Elev. 252', Revision 03  
 PFP-PWR43, Turbine Building - South / Elev. 252', Revision 05  
 PFP-PWR45, Turbine Building - North / Elev. 272', Revision 06  
 PFP-PWR46, Turbine Building - South / Elev. 272', Revision 04  
 PFP-PWR48, Turbine Building Elev. 300', Revision 04

**71111.07**Procedures

ST-2YA, Residual Heat Removal Heat Exchanger 'A' Performance Test, Revision 5

Drawings

4.95-5, RHR Heat Exchanger 'A' Tube Plugging Map 10E-2A, Revision 2

Miscellaneous

14620.9033, Total Available Tube Plugging Margin in the RHR Heat Exchanger at Related Conditions, Revision 0

**71111.11**Procedures

OP-65, Startup and Shutdown Procedures, Revision 123

OP-65B, Shutdown Operations, Revision 7

**71111.12**Issue Reports

04045089	04086665	04087668	04098168
04126033	04149166		

**71111.13**Procedures

AP-10.09, Outage Risk Assessment, Revision 37

**71111.15**Procedures

ISP 75-1, RCIC CST Low Water Level Switch Functional Test/Calibration, Revision 24

ER-AA-335-018, Visual Examination of ASME IWE Class MC and Metallic Liners of IWL Class CC Components, Revision 13

Issue Reports

04164618  
04164660  
04174135  
04178640

Work Orders

C04799709  
C04832920

Miscellaneous

Design Equivalent Change Package 625754

DWG 2.28-5, Sectional Allis Chalmers H16 x 12VTMC-3, Revision 003

Engineering Change (EC) 625691

**71111.18**Drawings

DWG 6.32-80, Model 7567F 6 x 10 Relief Valve Alternate Configuration, Revision 4

Miscellaneous

EC 620602, UFSAR Change Request Form per LA-AA-107-1001, Revision 7

EC 620602, Engineering Calculation, Evaluation of SRV QBUBS Load Changes on Torus Shell, Torus Attached Piping and Internal Structures, Revision 0

EC 620602, Engineering Calculation, S/RVDL Tailpipe Evaluation for ADS Line B, Revision 0

EC 620602, Engineering Calculation, S/RVDL Tailpipe Evaluation for Non-ADS Line L, Revision 0

ECN 624214, Target Rock ASME Design Report 8692, Revision D

JAF-RPT-11-00003, Operational Leakage Action Levels for Target Rock Three Stages Safety Relief Valves, Revision 1

JAF-SCN-18-056, 50.59 Screening Form for EC 620602, Revision 0

James A. FitzPatrick Nuclear Power Plant, Technical Proposal for 3-Stage Main Steam Safety Relief Valves

Report No. 8696, Temperature/Seat Leakage Correlation Report for Three Stage Pilot Operated 6x10 Safety Relief Valve SRV, Target Rock Model: 0867F-001, Revision D

Report TERI 081, Technical Evaluation of 3-Stage 6x10 MS-SRV Upgrade to Address Limited Flow Test Induced Defect (Model 0867F) Ref: Problem Report 080 and CAR 15-045, Revision A

**71111.19**Procedures

ISP-75-1, RCIC CST Low Water Level Switch Functional Test/Calibration, Revision 24

MP-200.03, Replacement of NAMCO EA-180 and EA-740 Position Switches, Revision 11

MST-029.02, Main Steam Line Isolation Valve 85 Percent Open (RPS) Position Switch Calibration Surveillance, Revision 19

ST-1CA, Outside Primary Containment Isolation Valve Exercise Test, Revision 1

ST-2AL, RHR Loop 'A' Quarterly Operability Test (IST), Revision 038

ST-76J23, West Diesel Fire Pump 76P-1 Performance Test, Revision 22

Issue Reports

04109193	04156252	04164618	04164660
04167340	04176976		

Work Orders

C04644984	C04732717	C80161193	C80403118
C82503567			

**71111.22**Procedures

IMP-71.25, Test Equipment Set Up for 10500 and 10600 Emergency Bus Sequencing Test ST- C, Revision 10

ST-1B, MSIV Fast Closure Test, Revision 26

ST-6HB, Standby Liquid Control Quarterly Operability Test (IST), Revision 9

ST-9BA, EDG 'A' and 'C' Full Load Test and ESW Pump Operability Test, Revision 16

ST-9CA, EDG 'A' and 'C' Load Sequencing Test and 4kV Emergency Power System Voltage  
Relays Instrument Functional Test, Revision 4  
ST-39B-X7A, Type 'C' Leak Test Main Steam Line 'A' MSIVs, Revision 16

Issue Reports  
04178600

Work Orders  
C82606584

### **71124.01**

#### Procedures

NISP-RP-004, Radiological Posting and Labeling, Revision 00  
NISP-RP-005, Access Controls for High Radiation Areas, Revision 0  
RP-AA-18, Radiological Posting and Labeling Program Description, Revision 1  
RP-AA-300, Radiological Survey Program, Revision 15  
RP-AA-460, Controls for High and Locked High Radiation Areas, Revision 34  
RP-AA-503, Unconditional Release Survey Method, Revision 14

#### Issue Reports

04171228	04174941	04175570	04175765
04176037	04176242		

#### RWPs/ALARA Plans

<u>RWP</u>	<u>Description</u>	<u>ALARA Plan</u>
JF-1-18-00506	DW Scaffold	AP-18-0506
JF-1-18-00510	DW SRV Activities	AP-18-0510
JF-1-18-00513	DW CRD Drive Exchange	AP-18-0513
JF-1-18-00516	DW RW Recirc Pump and Motor	AP-18-0516
JF-1-18-00541	DW 12RWC-46 and 12MOV-15	AP-18-0541

#### Surveys

<u>Survey</u>	<u>Description</u>	<u>Date/Time</u>
2018-021841	DW 268'	September 10, 2018/2100
2018-021911	DW 292' SRV	September 11, 2018/1254
2018-022001	Under Reactor Head	September 12, 2018/0052
2018-022623	12RWC-46	September 16, 2018/1058
2018-022711	12RWC-46	September 17, 2018/0837
2018-022565	SRV 71F	September 18, 2018/0116
2018-023578	DW 256'	September 25, 2018/0447

#### Air Sample Results

<u>Air Sample</u>	<u>Description</u>	<u>Date/Time</u>
18306	DW 12RWC46	September 18, 2018/1220
18320	DW 'B' Recirc Pump Seal	September 22, 2018/2300
18329	DW MSIV 'B' Breach	September 23, 2018/0250
18311	DW 12MOV15 Breach	September 23, 2018/1220



18342	DW 268' General Area	September 24, 2018/0640
18343	DW 292' General Area	September 24, 2018/0720
18345	DW 12MOV15	September 25, 2018/0230

**71124.02**Procedures

RP-AA-400, ALARA Program, Revision 15

Issue Reports (\*initiated in response to inspection)

04175408  
04177200\*

**71124.03**Procedures

RP-AA-301, Radiological Air Sampling Program, Revision 11  
RP-AA-302, Determination of Alpha Levels and Monitoring, Revision 10

Issue Reports

04133415

Miscellaneous

Breathing Air Samples, Report 294228-0 on January 18, 2017; and Report 341956-1 on August 31, 2018

**71124.04**Procedures

RP-AA-12, Internal Dose Control Program Description, Revision 2  
RP-AA-203-1001, Personnel Exposure Investigations, Revision 10  
RP-AA-210, Dosimetry Issue, Usage, and Control, Revision 29  
RP-AA-220, Bioassay Program, Revision 13  
RP-AA-222, Methods for Estimating Internal Exposure from in VIVO Bioassay Data, Revision 5  
RP-AA-290, Prenatal Radiation Exposure, Revision 8

Issue Reports

04102403  
04174125  
04175886

Miscellaneous

Generic Radiation Worker Training Lesson Plan, Revision 11, December 2017  
NVLAP Certification for Lab Code 100518-0

**71124.05**Procedures

ISP-95A, Post Accident Containment High Range Radiation Monitor A Functional Test/Calibration, Revision 4

ISP-95B, Post Accident Containment High Range Radiation Monitor B Functional Test/Calibration, Revision 4  
 RP-AA-700-1101, Calibration of the RO-2, RO-2A, RO-20 and RSO-50E Ion Chambers, Revision 1  
 RP-AA-700-1203, Calibration of the MGP Instruments Telepole, Revision 1  
 RP-AA-700-1215, Calibration of Low-Vol Air Samplers, Revision 2  
 RP-AA-700-1216, Calibration of Hi-Vol Air Samplers, Revision 4  
 RP-AA-700-1235, Operation and Calibration of the PM-12 Gamma Portal Monitor, Revision 5  
 RP-AA-700-1239, Operation and Calibration of the Model SAM-12 Small Articles Monitor, Revision 4  
 RP-AA-700-1240, Operation and Calibration of the Canberra ARGOS-5 Personnel Contamination Monitor, Revision 6  
 RP-AA-700-1242, Calibration of Eberline Model 6112B Teletector, Revision 0  
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