



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 1, 2018

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 –
CORRECTION TO AMENDMENT NOS. 321 AND 324 TO ADOPT
10 CFR 50.69, "RISK-INFORMED CATEGORIZATION AND TREATMENT OF
STRUCTURES, SYSTEMS AND COMPONENTS FOR NUCLEAR POWER
REACTORS" (CAC NOS. MG0181 AND MG0182; EPID L-2017-LLA-0281)

Dear Mr. Hanson:

On October 25, 2018 (Agencywide Documents Access and Management System Accession No. ML18263A232), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment Nos. 321 and 324 to Renewed Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station (Peach Bottom), Units 2 and 3, respectively. These amendments were issued in response to your application dated August 30, 2017, as supplemented by letters dated October 24, 2017; May 7, 2018; June 6, 2018; August 10, 2018; and August 22, 2018.

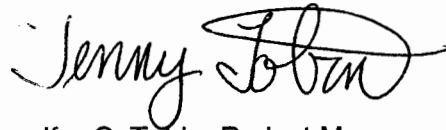
These amendments added a new license condition to the Renewed Facility Operating Licenses to allow the implementation of risk-informed categorization and treatment of structures, systems, and components for nuclear power reactors in accordance with Title 10 of the *Code of Federal Regulations* Section 50.69.

Shortly after these amendments were issued, it was noted that there was an inadvertent typographical error on license page 7g for Unit 3. Rather than the language in paragraph (17) on license page 7g for Unit 3 referring to License No. DPR-56 for Peach Bottom Unit 3, it referred to License No. DPR-44 for Peach Bottom Unit 2. The corrected page 7g is enclosed with a change bar showing the line of text that has been corrected.

Please replace page 7g of Renewed Facility Operating License No. DPR-56 for Unit 3 with the enclosed page. The NRC staff regrets any inconvenience this may have caused.

If you have any questions, please contact me at 301-415-2328 or Jennifer.Tobin@nrc.gov.

Sincerely,

A handwritten signature in black ink, reading "Jennifer C. Tobin". The signature is fluid and cursive, with the first name "Jennifer" and last name "Tobin" clearly legible.

Jennifer C. Tobin, Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-277 and 50-278

Enclosure:
Corrected page 7g for Renewed DPR-56

cc: Listserv

Enclosure

Corrected page 7g for Renewed DPR-56

- (e) The results of the power ascension testing to verify the continued structural integrity of the steam dryer shall be submitted to the NRC staff in a report in accordance with 10 CFR 50.4. The report shall include a final load definition and stress report of the steam dryer, including the results of a complete re-analysis using the end-to-end B/Us from Peach Bottom Unit 2 benchmarking at EPU conditions. The report shall be submitted within 90 days of the completion of EPU power ascension testing for Peach Bottom Unit 3.
- (f) During the first two scheduled refueling outages after reaching EPU conditions, a visual inspection shall be conducted of the steam dryer as described in the inspection guidelines contained in WCAP-17635-P.
- (g) The results of the visual inspections of the steam dryer shall be submitted to the NRC staff in a report in accordance with 10 CFR 50.4. The report shall be submitted within 90 days following startup from each of the first two respective refueling outages.
- (h) Within 6 months following completion of the second refueling outage, after the implementation of the EPU, the licensee shall submit a long-term steam dryer inspection plan based on industry operating experience along with the baseline inspection results.

The license condition described above shall expire: (1) upon satisfaction of the requirements in paragraphs (f) and (g), provided that a visual inspection of the steam dryer does not reveal any new unacceptable flaw(s) or unacceptable flaw growth that is due to fatigue, and; (2) upon satisfaction of the requirements specified in paragraph (h).

(16) Maximum Extended Load Line Limit Analysis Plus (MELLLA+) Special Consideration

The licensee shall not operate the facility within the MELLLA+ operating domain with a feedwater heater out of service resulting in more than a 10°F reduction in feedwater temperature below the design feedwater temperature.

(17) Adoption of 10 CFR 50.69, "Risk-informed categorization and treatment of structures, systems, and components for nuclear power plants"

In support of implementing License Amendment No. 321 permitting the adoption of the provisions of 10 CFR 50.69 for Renewed Facility Operating License No. DPR-56 for Peach Bottom Unit 3, the license is amended to add the following license condition:

Renewed License No. DPR-56
Amendment No. 324

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DATED NOVEMBER 1, 2018

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