

NRC-DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO: N.C. MOSELEY

FROM: TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENN.
H.S. FOXDATE OF DOCUMENT
10-19-76DATE RECEIVED
10-29-76☐ LETTER
☒ ORIGINAL
☐ COPY☐ NOTORIZED
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1

DESCRIPTION

LICEN LTR. TRANS THE FOLLOWING....

ENCLOSURE

LICENSEE EVENT REPORT FOR R.O.# 7613, on 9-20-76
CONCERNING MAIN STEAM LINE "D" INDICATOR FI46-4'
AND DIFFERENTIAL PRESSURE INDICATING SWITCHES.
PDIS 1-50(a-d) BEING FOUND UNRESPONSIVE TO
STEAM FLOW.....(1 SIGNED CY. RECEIVED)
(3 PAGES)

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

PLANT NAME: BROWNS FERRY # 3

FOR ACTION/INFORMATION

SAB 11-3-76

☒ BRANCH CHIEF: STOLZ
☒ W/3 CYS FOR ACTION
☒ LIC. ASST.: LEE
☒ W/ CYS
☒ ACRS 16 CYS ~~XXXXXXXX~~ SENT TO ~~XX~~ ACRS CAT "A"

INTERNAL DISTRIBUTION

☒ REG FILE
☒ NRC PDR
☒ I & E (2)
☒ MIPC
☒ SCHROEDER/IPPOLITO
☒ HOUSTON
☒ NOVAK/CHECK
☒ GRIMES
☒ CASE
☒ JEFFER
☒ HANAUER
☒ TEFESCO/MACCARY
☒ EISENHUT
☒ PAER
☒ SHAO
☒ VOLLMER/FUNCH
☒ KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

☒ TADR: ATHENS, ALA
☒ TIC:
☒ NSIC:

CONTROL NUMBER

10919



THE
UNITED STATES
DEPARTMENT OF
THE ARMY
WASHINGTON, D. C.

TO THE
HONORABLE
THE SECRETARY OF THE ARMY
WASHINGTON, D. C.

FROM
THE
HONORABLE
THE SECRETARY OF THE ARMY
WASHINGTON, D. C.

SUBJECT
THE
UNITED STATES
DEPARTMENT OF
THE ARMY
WASHINGTON, D. C.



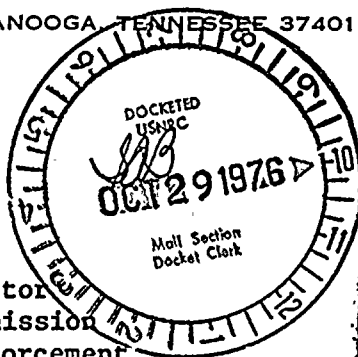
Regulatory

File Cy-7

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

October 19, 1976



Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., 8th Floor
Atlanta, Georgia 30303

Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 -
DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE
OCCURRENCE REPORT BFRO-50-296/7613

The enclosed report is to provide details concerning the main steamline D flow indicator FI-46-4 and differential pressure indicating switches Pdis-1-50 (A-D) that were found unresponsive to steamflow during startup testing at 270 MWe. This report is submitted in accordance with Browns Ferry Technical Specifications Section 6. This event occurred on Browns Ferry Nuclear Plant unit 3.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure (3)

CC (Enclosure):

Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

10919



MEMORANDUM FOR THE DIRECTOR

RE: [Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]

DATE: [Illegible]

TO: [Illegible]
FROM: [Illegible]
SUBJECT: [Illegible]

[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]

BY: [Illegible]

DATE: [Illegible]

[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]

(S) [Illegible]
[Illegible]

[Illegible text]
[Illegible text]
[Illegible text]
[Illegible text]

(S) [Illegible]
[Illegible text]
[Illegible text]
[Illegible text]

LICENSEE EVENT REPORT

[PLEASE PRINT ALL REQUIRED INFORMATION]

BLOCK: 1 5 1 1 1 6

LICENSEE NAME														LICENSE NUMBER											LICENSE TYPE					EVENT TYPE								
L B R F 3														0 0 - 0 0 0 0 0 - 0 0											4 1 1 1 1					0 3								
14														15											25					26					30		31 32	

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER											EVENT DATE					REPORT DATE																			
01 CONT		L		L		0 5 0 - 0 2 9 6											0 9 2 0 7 6																								
7 8		57 58		59 60		61											68					69					74					75					80				

EVENT DESCRIPTION

02 During startup testing at 270 MWe, main steam line "D" flow indicator FI-46-4 and
 03 differential pressure indicating switches Pdis-1-50 (A-D) were found unresponsive to
 04 steam flow. As a result "D" main steam line was isolated. Sensing lines B4 and B6
 05 were reversed at panel 25-56 to resolve problem and subsequent proper response to
 06 steam flow was confirmed. (BFRO-50-296/7613)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION			
07 C D		A		I N S T R U				N		B 0 8 0				N			
7 8 9 10		11		12				43		44				47		48	

CAUSE DESCRIPTION

08 Low pressure sensing line B4 was switched in the drywell penetration with the
 09 recirculation jet pump riser low pressure sensing line B6.
 10

FACILITY STATUS		% POWER				OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
11 B		0 2 4				NA				C		Review of surveillance data					
7 8 9		10 12 13				44				45		46					

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE					
12 Z		Z		NA				NA					
7 8 9		10 11		44				45					

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION					
13 0 0 0		Z		NA					
7 8 9		11 12		13					

PERSONNEL INJURIES

NUMBER		DESCRIPTION					
14 0 0 0		NA					
7 8 9		11 12					

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION					
16 Z		NA					
7 8 9		10					

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 See attachment.
 19

NAME: _____

PHONE: _____

GPO 821-647

BFRO-50-296/7613 (Continued)

Additional Factors

All others similar instrument NA lines were checked and found correctly installed. The LPCI logic was not made inoperable due to the lines been reversed. Two parallel sets of contacts in each loop provide redundancy.

BFRO-50-296/7613 (Continued)

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