NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

DOCKET NUMBER: 50-[redacted]296

FILE NUMBER: INCIDENT REPORT

TO:	FROM:	DATE OF DOCUMENT
Norman C. Moseley	Tennessee Valley Authority Chattanooga, Tenn H. S. Fox	7/14/77
		DATE RECEIVED: 7/18/77

X LETTER ☐ ORIGINAL X COPY	☐ NOTORIZED X UNCLASSIFIED	PROP	INPUT FORM	NUMBER OF COPIES RECEIVED: 1 CC

DESCRIPTION

DO NOT REMOVE

ACKNOWLEDGED

PLANT NAME: Browns Ferry Nuclear Plant
Unit No. 3
RBT 7/19/77

ENCLOSURE

Licensee Event Report (50-296/77-8) on 6/21/77 concerning the releif valve on the discharge of standby liquid control pump 3A which opened at low pressure and stuck in a partially open position during conduct of SI 4.4.A.1.
1p++5p

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

1 cy ENCL Rec'd

FOR ACTION/INFORMATION

BRANCH CHIEF:	Schwencer
W/ 3 CYS FOR ACTION	
LIC ASST.:	Sheppard

INTERNAL DISTRIBUTION

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NRC PDR
I & E (2)
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SCHROEDER/IPPOLITO
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EISENHUT
BAER
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VOLLMER/BUNCH
KREGER/ J. COLLINS
ROSA

EXTERNAL DISTRIBUTION

LPDR: Athens, ALA.
TIC:
NSIC:
ACRS (16) SENT AS CAT. B

CONTROL NUMBER

AD
4 [redacted]

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

JUL 14 1977

Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., Suite 1217
Atlanta, Georgia 30303

Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFRO-50-296/778

The enclosed report is to provide details concerning the relief valve on the discharge of standby liquid control pump 3A which was observed to open at 1300 psig instead of the designed setting of 1400 psig and which stuck in a partially open position during performance of SI 4.4.A.1. This report is submitted in accordance with Browns Ferry Technical Specifications Section 6. This event occurred on Browns Ferry unit 3.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure (3)
cc (Enclosure):
Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6)

(PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	ALBRF3 (7–14)	00–00000–00 (15–25)	4111 1 (26–30)	03 (31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CON'T	(57–58)	L (59)	L (60)	050–0296 (61–68)	062177 (69–74)	(75–80)

EVENT DESCRIPTION

02 (7–9) (80)

03 (7–9) (80)

04 (7–9) (SEE ATTACHED) (80)

05 (7–9) (80)

06 (7–9) (80)

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	SF (10)	E (11)	VALVEX (12–17)	N (43)	C710 (44–47)	N (48)

CAUSE DESCRIPTION

08 (7–9) (80)

09 (7–9) (SEE ATTACHED) (80)

10 (7–9) (80)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	E (9)	065 (10–12)	NA (13–44)	B (45)	NA (46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z (9)	Z (10)	NA (11–44)	NA (45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000 (9–11)	Z (12)	NA (13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000 (9–11)	NA (12–80)

OFFSITE CONSEQUENCES

15 NA (7–9) (80)

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z (9)	NA (10–80)

PUBLICITY

17 NA (7–9) (80)

ADDITIONAL FACTORS

18 NA (7–9) (80)

19 NA (7–9) (80)

NAME:_____ PHONE:_____

GPO 881-667

NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

July 13, 1977

Mr J G Keppler, Director, Region III
Office of Inspection & Enforcement
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Mr Keppler:

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket No. 50-282 License No. DPR-42

Inoperability of #12 AFWP Discharge Valve

The Licensee Event Report for this occurrence is reproduced on the back of this letter. Enclosed are three copies.

Yours very truly,

L K Mayer for

L O Mayer, PE
Manager of Nuclear Support Services

LOM/LLT/deh

cc: Director, IE, USNRC (30)
Director, MIPC, USNRC (3)
G Charnoff
MPCA
Attn: J W Ferman

-over-

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6)

(PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	MNPIN1 (7–8, 9–14)	00–00000–00 (15–25)	4111 1 (26–30)	03 (31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CON'T	(57–58)	L (59)	L (60)	050–0282 (61–68)	061877 (69–74)	071377 (75–80)

EVENT DESCRIPTION

02 During hot shutdown, while feeding the steam generators with No. 12 Auxiliary
03 Feedwater Pump, the pump discharge valve motor operator failed. The redundant
04 pump was started to supply feedwater. Not a repetitive occurrence.
05
06 (P-RO-77-24)

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	CH (10)	E (11)	VALVOP (12–17)	A (43)	L200 (44–47)	(48)

CAUSE DESCRIPTION

08 Open lead in limitorque SMB-000 motor operator caused by rubbing (believed to be
09 a random failure). Lead was repaired and valve tested for operability.
10

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	G (9)	000 (10–12)	NA (13–44)	A (45)	NA (46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z (9)	Z (10)	NA (11–44)	NA (45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000 (9–11)	Z (12)	NA (13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000 (9–11)	NA (12–80)

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z (9)	NA (10–80)

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 NA

19

NAME: A. A. Hunstad PHONE: 612-388-4767

GPO 861-667

Event Description

During performance of SI 4.4.A.1, the relief valve on the discharge of standby liquid control pump 3A was observed to open at 1300 psig instead of the designed setting of 1400 psig and stuck in a partially open position. The relief valve was removed from the system, disassembled, and the plug, plug guide, and seat were replaced. The valve was tested and was observed to open at 1400 psig. It was then re-installed in the system, and SI 4.4.A.1 was performed satisfactorily. All redundant systems were available. A similar event occurred on this same pump on January 22, 1977, when another relief valve which was subsequently replaced, also opened prematurely. (BFRO-50-296/778)

Cause Description

The inside surface of the plug guide and outside surface of the plug both had a scored area of approximately one-quarter inch square. The scored places caused the relief valve to stick in the open position. It is not known what caused the valve to open at a pressure lower than designed; because after the valve was removed from the system, before parts were replaced, it was tested and found to open at 1400 psig. It is possible that during the throttling process there was a pressure spike causing the valve to lift at its set pressure and stick in the open position. The scoring appears to have been caused by foreign material which worked in the small clearance between the plug and plug guide. The relief valve is a "Crosby" 1 x 2 style JMWK. Replacement parts were installed as indicated in the "Event Description".

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

July 14, 1977

Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., Suite 1217
Atlanta, Georgia 30303

Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFRO-50-296/778

The enclosed report is to provide details concerning the relief valve on the discharge of standby liquid control pump 3A which was observed to open at 1300 psig instead of the designed setting of 1400 psig and which stuck in a partially open position during performance of SI 4.4.A.1. This report is submitted in accordance with Browns Ferry Technical Specifications Section 6. This event occurred on Browns Ferry unit 3.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure (3)
cc (Enclosure):
Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, N.W., Suite 1217
Atlanta, Georgia 30303

Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 -
DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE
OCCURRENCE REPORT BFRO-50-296/7718

The enclosed report is to provide details concerning the relief valve on the discharge of standby liquid control pump 3A which was observed to open at 1300 psig instead of the designed setting of 1400 psig and which acted in a partially open position during performance of SI 4.4.A.1. This report is submitted in accordance with Browns Ferry Technical Specifications Section 6. This event occurred on Browns Ferry unit 3.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

S. Fox
Director of Power Production

Enclosure (3)
cc (Enclosure):
Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6)

(PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
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	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
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EVENT DESCRIPTION

02 (7–80)

03 (7–80)

04 (SEE ATTACHED) (7–80)

05 (7–80)

06 (7–80)

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
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CAUSE DESCRIPTION

08 (7–80)

09 (SEE ATTACHED) (7–80)

10 (7–80)

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	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
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PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000 (9–11)	Z (12)	NA (13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000 (9–11)	NA (12–80)

OFFSITE CONSEQUENCES

15 NA (7–80)

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z (9)	NA (10–80)

PUBLICITY

17 NA (7–80)

ADDITIONAL FACTORS

18 NA (7–80)

19 NA (7–80)

NAME: _____ PHONE: _____

GPO 881-667

Event Description

During performance of SI 4.4.A.1, the relief valve on the discharge of standby liquid control pump 3A was observed to open at 1300 psig instead of the designed setting of 1400 psig and stuck in a partially open position. The relief valve was removed from the system, disassembled, and the plug, plug guide, and seat were replaced. The valve was tested and was observed to open at 1400 psig. It was then re-installed in the system, and SI 4.4.A.1 was performed satisfactorily. All redundant systems were available. A similar event occurred on this same pump on January 22, 1977, when another relief valve which was subsequently replaced, also opened prematurely. (BFRO-50-296/778)

Cause Description

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