n \$ 102/198

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) DISTRIBUTION FOR INCOMING MATERIAL 50-296

REC: OREILLY J P ORG: FOX H S NRC TN VALLEY AUTH

DOCDATE: 07/25/78 DATE RCVD: 07/28.78

DOCTYPE: LETTER NOTARIZED: NO SUBJECT:

COPIES RECEIVED LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-296/78-019) ON 07/07/78 CONCERNING EXCESSIVE DRYWELL FLOOR DRAIN LEAK RATE OBSERVED, CAUSED BY PARTIALLY BROKEN WELD ON 1-INCH SOCKET WELD FITTING ON INSTRUMENT SENSING LINE ON A JET PUMP RISER.

PLANT NAME: BROWNS FERRY - UNIT 3

REVIEWER INITIAL: XJM' DISTRIBUTOR INITIAL:

INCIDENT REPORTS (DISTRIBUTION CODE A002)

FOR ACTION:

BR CHIEF ORB#3 BC\*\*W/4 ENCL

INTERNAL:

REG FILE\*#W/ENCL <u>I & G\*#W/2</u> ENCL I & C SYSTEMS BR\*\*W/ENCL NOVAK/CHECK\*\*W/ENCL AD FOR ENG\*\*W/ENCL HANAUER\*\*W/ENCL AD FOR SYS & PROJ\*\*W/ENCL ENGINEERING BR\*\*W/ENCL KREGER/J. COLLINS\*\*W/ENCL K SEYFRIT/IE\*\*W/ENCL

NRC PDR\*\*W/ENCL MIPC\*\*W/3 ENCL EMERGENCY PLAN BR\*\*W/ENCL EEB\*\*W/ENCL PLANT SYSTEMS BR\*\*W/ENCL AD FOR PLANT SYSTEMS\*\*W/ENCL REACTOR SAFETY BR\*\*W/ENCL VOLLMER/BUNCH\*\*W/ENCL POWER SYS BR\*\*W/ENCL

EXTERNAL:

LPDR'S ATHENS, AL\*\*W/ENCL TIC, LIZ CARTER\*\*W/ENCL NSIC\*\*W/ENCL ACRS CAT B\*\*W/16 ENCL

AJOY

LTR 45 ENCL 45 DISTRIBUTION: SIZE: 1P+1P

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THE END

CONTROL NBR: 782140087

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## TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

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July 25, 1978

Mr. James P. O'Reilly, Director U.S. Nuclear Regulatory Commission Office of Inspection and Enforcement Region II 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFR0-50-296/7819

The enclosed report provides details concerning an excessive drywell floor drain leak rate which was observed by the operator during normal operation. This report is submitted in accordance with Browns Ferry unit 3 technical specification 6.7.2.a.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox Director of Power Production

Enclosure (3) cc (Enclosure):

Director (3)

Office of Management Information and Program Control U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Director (40) Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, D.C. 20555 , , , , , 

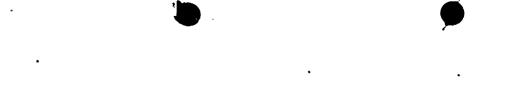
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NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7-77) LICENSEE EVENT REPORT ..... CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) (1)30 0 0 0 0 0 L В R F ](3) 0 0 (4) 00 (5) LICENSEE CODE LICENSE NUMBER CON'T REPORT 96000707 0 1 0 0 0 2 18 (8) 0 71 2 5 <u>L (6)</u> SOURCE 68 EVENT DATE REPORT DATE DOCKET NUMBER 69 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) excessive drywell floor drain leak rate was 0 2 During normal operation observed by the operator (T.S.3.6.C.1). The reactor was manually placed in cold shutdown within the 0 3 24 hour T.S. time limit (T.S.3.6.C.3). There was no effect on the public health 04 safety. Redundant systems were available had the instrumentation become inoperable 0 5 The leakage did not affect connected 0 6 instrumentation 0 7 0 8 80 SYSTEM CAUSE CAUSE SUBCODE COMP. VALVE SUBCODE CODE COMPONENT CODE SUBCODE C | B | (11 B 13 E X | X |(14 (12) 1 P E Z (16) 9 P A (15) 10 12 13 11 18 19 20 SEQUENTIAL OCCURRENCE REPORT REVISION EVENT YEAR REPORT NO. LER/RO CODE TYPE NO. 18 (17) REPORT 0 1 9 T 0 1 0 NUMBER 22 26 27 28 30 31 32 ACTION .FUTURE TAKEN ACTION EFFECT ON PLANT SHUTDOWN METHOD ATTACHMENT SUBMITTED NPRD-4 COMPONENT PRIME COMP. HOURS (22) FORM SUB. MANUFACTURER SUPPLIER A (21) <u>Y</u> (23) (20) 0 4 8 N (24 19 A 01 L (25) Х 9 (26) 36 35 42 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) 1 0 A partially broken weld on a 1-inch socket weld fitting on an instrument sensing  $\Pi$ jet pump riser was the cause of the leakage lon a The. line was 8 class stainless steel pipe pressurized to reactor pressure. The line was rewelded and the unit re-1 2 Other sensing lines of a similar configuration were inspected to [turned to service. 1 3 prevent recurrence. Design evaluation of the problem has been requested 14 9 8 80 FACILITY METHOD OF (30) DISCOVERY DESCRIPTION (32) % POWER OTHER STATUS E (28) 0 0 29 A (31) 5 N/A 11 Operator Observation ACTIVITY CONTENT 44 45 46 80 AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE RELEASED\_OF RELEASE Z 33 Z 34 6 N/A N/A 10 11 PERSONNEL EXPOSURES 10 44 45 80 DESCRIPTION (39) NUMBER TYPE 0 0 0 37 Z 38 7 N/A PERSONNEL INJURIES 80 DESCRIPTION (41) NUMBER 40 N/A 8 0 0 0 1 9 11 12 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE (42) No damage - Temporary use of unit was lost 9 L 10 80 PUBLICITY NRC USE ONLY ISSUED DESCRIPTION (45) 91 7-92 0 N/A 8 9 10 68 69 80 NAME OF PREPARER. PHONE:-



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