UNITED STATES NUCLEAR REGULATORY COMMISSION
PRESSURIZED WATER REACTOR GENERIC FUNDAMENTALS EXAMINATION
MARCH 2018 PWR – FORM A

Please Print

Name: _____

Docket No.: _____

Facility: _____

Start Time: _____ Stop Time: _____

INSTRUCTIONS TO EXAMINEE

Answer all the test items using the answer sheet provided, ensuring a single answer is marked for each test item. Each test item has equal point value. A score of at least 80 percent is required to pass this portion of the NRC operator licensing written examination. All examination materials will be collected 3 hours after the examination begins. This examination applies to a typical U.S. pressurized water reactor (PWR) nuclear power plant.

SECTION	QUESTIONS	% OF TOTAL	SCORE
COMPONENTS	1 - 22		
REACTOR THEORY	23 - 36		
THERMODYNAMICS	37 - 50		
TOTALS	50		

All work performed on this examination is my own. I have neither given nor received aid.

_____

Examinee's Signature

RULES AND INSTRUCTIONS FOR THE
NRC GENERIC FUNDAMENTALS EXAMINATION

During the administration of this examination the following rules apply:

NOTE: The term "control rod" refers to the length of neutron absorber material that can be positioned by the operator to change core reactivity.

NOTE: Numerical answers are rounded to the nearest whole number unless otherwise indicated.

1. Print your name in the blank provided on the cover sheet of the examination.

2. Fill in your individual docket number.

3. Fill in the name of your facility.

4. Fill in your start and stop times at the appropriate times.

5. Two aids are provided for your use during the examination:

   (1) An Equations and Conversions Sheet contained within the examination copy, and

   (2) Steam tables and Mollier Diagram provided by your proctor.

6. Place your answers on the answer sheet provided. Credit will only be given for answers properly marked on this sheet. Follow the instructions for filling out the answer sheet.

7. Scrap paper will be provided for calculations.

8. Cheating on the examination will result in the automatic forfeiture of this examination. Cheating could also result in severe penalties.

9. Restroom trips are limited. Only **one** examinee may leave the room at a time. In order to avoid the appearance or possibility of cheating, avoid all contact with anyone outside the examination room.

10. After you have completed the examination, sign the statement on the cover sheet indicating that the work is your own and you have neither given nor received any assistance in completing the examination. Either pencil or pen may be used.

11. Turn in your examination materials, answer sheet on top, followed by the examination copy and the examination aids, e.g., steam tables, handouts, and scrap paper.

12. After turning in your examination materials, leave the examination area as defined by the proctor. If after leaving you are found in the examination area while the examination is in progress, your examination may be forfeited.

GENERIC FUNDAMENTALS EXAMINATION
EQUATIONS AND CONVERSIONS SHEET

EQUATIONS

$\dot{Q} = \dot{m}c_p\Delta T$

$\dot{Q} = \dot{m}\Delta h$

$\dot{Q} = UA\Delta T$

$\dot{Q} \propto \dot{m}^3_{\text{Nat Circ}}$

$\Delta T \propto \dot{m}^2_{\text{Nat Circ}}$

$K_{eff} = 1/(1 - \rho)$

$\rho = (K_{eff} - 1)/K_{eff}$

$SUR = 26.06/\tau$

$$\tau = \frac{\bar{\beta}_{eff} - \rho}{\lambda_{eff}\,\rho}$$

$$\rho = \frac{\ell^*}{\tau} + \frac{\bar{\beta}_{eff}}{1 + \lambda_{eff}\,\tau}$$

$\ell^* = 1.0 \times 10^{-4}$ sec

$\lambda_{eff} = 0.1\ \text{sec}^{-1}$ (for small positive ρ)

$DRW \propto \varphi^2_{tip} / \varphi^2_{avg}$

$P = P_o e^{t/\tau}$

$P = P_o 10^{SUR(t)}$

$A = A_o e^{-\lambda t}$

$N = S/(1 - K_{eff})$

$CR_1(1 - K_{eff_1}) = CR_2(1 - K_{eff_2})$

$1/M = CR_1/CR_x$

$A = \pi r^2$

$F = PA$

$\dot{m} = \rho A\vec{v}$

$\dot{W}_{Pump} = \dot{m}\Delta P\upsilon$

$P = I^2R$

$P = IE$

$P_A = \sqrt{3}IE$

$P_T = \sqrt{3}IEpf$

$P_R = \sqrt{3}IE\sin\theta$

Thermal Efficiency = Net Work Out/Energy In

$$\frac{g(z_2 - z_1)}{g_c} + \frac{(\vec{v}_2^2 - \vec{v}_1^2)}{2g_c} + \upsilon(P_2 - P_1) + (u_2 - u_1) + (q - w) = 0$$

$g = 32.2\ \text{ft/sec}^2$

$g_c = 32.2$ lbm-ft/lbf-sec^2

CONVERSIONS

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1 MW $= 3.41 \times 10^6$ Btu/hr

1 hp $= 2.54 \times 10^3$ Btu/hr

1 Btu $= 778$ ft-lbf

°C $= (5/9)(°F - 32)$

°F $= (9/5)(°C) + 32$

1 kg $= 2.21$ lbm

1 ft^3_{water} $= 7.48$ gal

1 gal_{water} $= 8.35$ lbm

1 Curie $= 3.7 \times 10^{10}$ dps

QUESTION: 1

A completely full water storage tank is being hydrostatically tested to 200 psig using a positive displacement pump (PDP) with a smooth and constant discharge flow rate of 8 gpm. The tank is protected by a relief valve and a safety valve that both discharge to the atmosphere. The valves have the following characteristics:

- The relief valve opening setpoint is 200 psig with an accumulation of 5 percent.
- The safety valve opening setpoint is 240 psig with a blowdown of 5 percent.
- Both valves have a maximum discharge flow rate of 6 gpm.

The PDP is inadvertently left running when tank pressure reaches 200 psig.

When conditions stabilize with the PDP still running, the relief valve will be _____ open; and the safety valve will be discharging a flow rate of approximately _____ to the atmosphere.

A. fully; 2 gpm

B. fully; 6 gpm

C. partially; 2 gpm

D. partially; 6 gpm

QUESTION: 2

Refer to the drawing of a valve (see figure below).

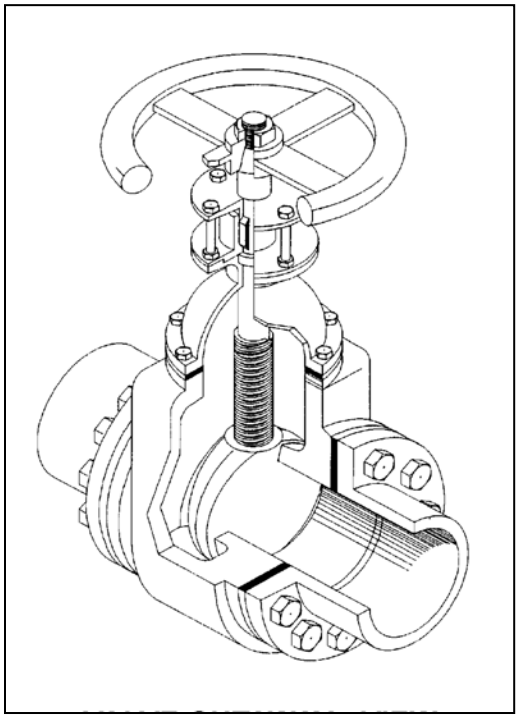
Which one of the following describes the type of valve shown?

A. Rising-stem globe valve

B. Nonrising-stem globe valve

C. Rising-stem gate valve

D. Nonrising-stem gate valve



QUESTION: 3

Refer to the drawing of a differential pressure (D/P) level detection system for a pressurizer at normal operating temperature and pressure (see figure below).

A nuclear power plant uses several differential pressure detectors like the one below to provide multiple channels of pressurizer water level indication. A hot channel was calibrated when the pressurizer was at normal operating temperature. A cold channel was calibrated when the pressurizer was at 160°F.

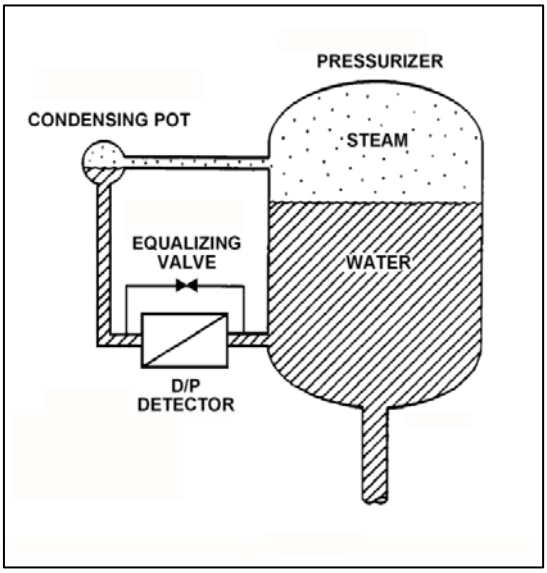
How will the level indications on the two channels compare when the pressurizer is at normal operating temperature?

A. The cold channel will indicate higher than the hot channel, due to the difference in reference leg water density at the two calibration temperatures.

B. The cold channel will indicate lower than the hot channel, due to the difference in reference leg water density at the two calibration temperatures.

C. The cold channel will indicate higher than the hot channel, due to the difference in pressurizer water density at the two calibration temperatures.

D. The cold channel will indicate lower than the hot channel, due to the difference in pressurizer water density at the two calibration temperatures.



QUESTION: 4

A cooling water system bourdon tube pressure detector is located inside a sealed building and system pressure currently indicates 50 psig. A building ambient temperature increase of 20°F will cause a _____ change in indicated system pressure; a building pressure increase of 20 psig will cause a _____ change in indicated system pressure.

A. significant; significant

B. negligible; significant

C. significant; negligible

D. negligible; negligible

QUESTION: 5

Refer to the drawing of a simple chromel-alumel thermocouple circuit (see figure below). Initially, the temperature instrument indicates 350°F.

A steam leak inside the reactor building (RB) increases the temperature of the thermocouple connection head, reference junction panel, and extension wires inside the RB from 120°F to 160°F. The temperature at the location measured by the thermocouple remains the same.

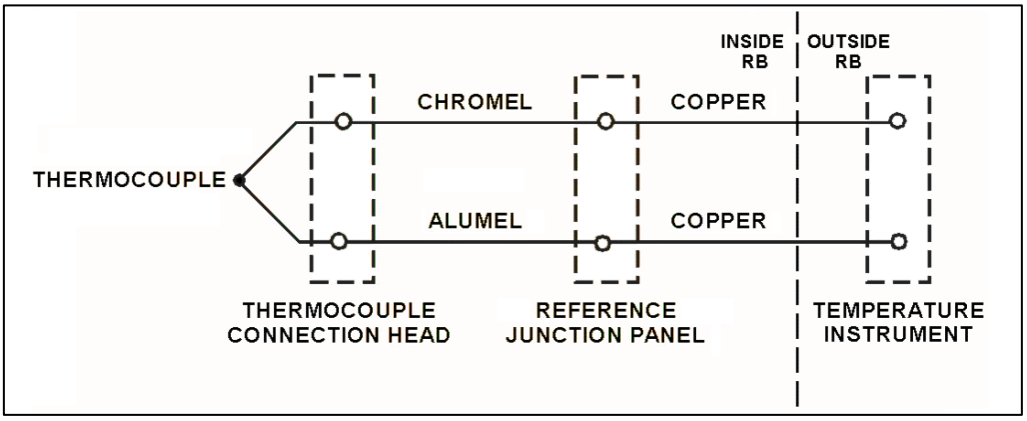
What is the resulting temperature indication?

A. 310°F

B. 350°F

C. 390°F

D. 430°F



QUESTION: 6

A nuclear plant worker normally wears a thermoluminescent dosimeter (TLD) or similar device for measuring whole body radiation exposure. When a self-reading pocket dosimeter (SRPD) is also required for whole body monitoring, where will the SRPD be worn and why?

A. Near the TLD to add exposure to the TLD measurement.

B. Near the TLD to measure radiation affecting the same part of the body.

C. Away from the TLD to add exposure to the TLD measurement.

D. Away from the TLD to measure radiation affecting a different part of the body.

QUESTION: 7

Refer to the drawing of a pressure alarm circuit (see figure below). The orientation of the bistable symbol indicates the characteristics of the bistable, as is normal for a control circuit diagram.

The bistable turns on to actuate an alarm at a system pressure of 100 psig. The bistable has a 5 psig deadband, or neutral zone.

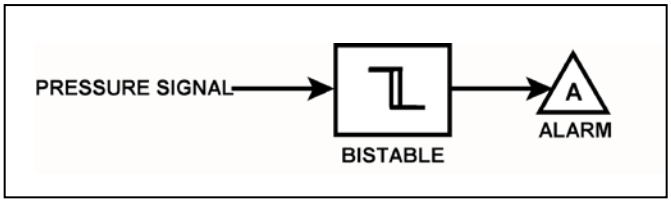
If system pressure is currently 110 psig, which one of the following describes the alarm circuit response as system pressure slowly decreases to 90 psig?

A. The alarm will actuate at 100 psig and will <u>not</u> turn off.

B. The alarm will actuate at 100 psig and will turn off at 95 psig.

C. The alarm is currently actuated and will <u>not</u> turn off.

D. The alarm is currently actuated and will turn off at 95 psig.



QUESTION: 8

Refer to the drawing of a water storage tank with an automatic level control system (see figure below). The level control system uses a reverse-acting level sensor and a direct-acting controller. The flow rate of water entering the tank is constant, and within the capacity of the drain valve.

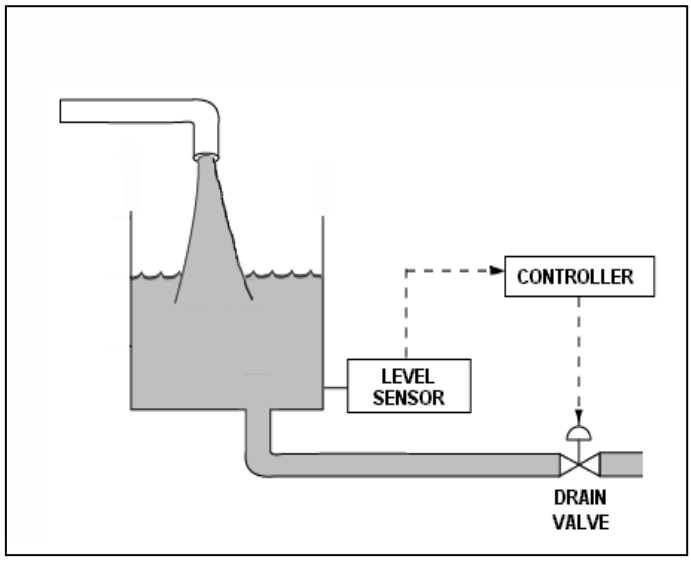
For the level control system to maintain a stable water level in the tank at a value up to 10 percent above or below the controller's setpoint, the controller must have a _____ characteristic; and the drain valve must fail _____ on a loss of air pressure to its actuator.

A. proportional-only; closed

B. proportional-only; open

C proportional-integral; closed

D. proportional-integral; open



QUESTION: 9

The level in a condensate collection tank is being controlled by an automatic level controller using proportional-only control. Initially the tank level is stable, but then the flow into the tank increases and stabilizes at a higher flow rate.

As tank level increases, the controller positions a drain valve more open than necessary to stabilize the level. As tank level decreases, the controller positions the drain valve more closed than necessary to stabilize the level. This cycle is repeated continuously, never reaching a stable tank level or drain valve position.

The excessive valve positioning described above could be caused by the controller's gain being too _____; or by the controller's proportional band being too _____.

A. low; wide

B. low; narrow

C. high; wide

D. high; narrow

QUESTION: 10

Refer to the drawing of a radial-flow centrifugal pump with a recirculation line in an open system (see figure below). The recirculation line throttle valve is currently 50 percent open. The pump is currently operating very close to runout.

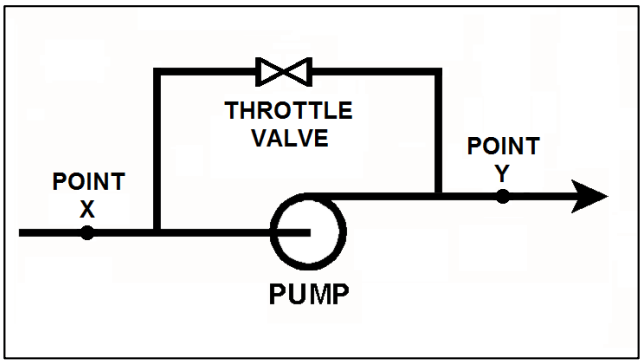
To move pump operation farther away from runout, without reducing the pump's available net positive suction head, an orifice can be installed at point _____; or the pump's recirculation line throttle valve can be positioned more _____.

A. X; open

B. X; closed

C. Y; open

D. Y; closed



QUESTION: 11

Refer to the drawing of a lube oil temperature control system (see figure below).

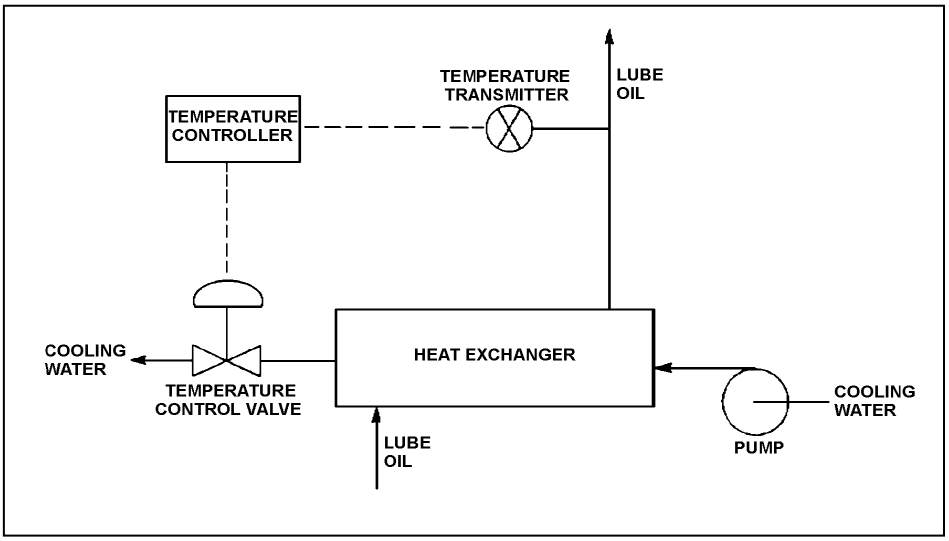
Initially, the pump is operating with the temperature control valve one-half open. If the temperature control valve is positioned more closed, the system head loss will _____; and the pump head will _____.

A. increase, increase

B. increase, decrease

C. decrease, increase

D. decrease, decrease



QUESTION: 12

Refer to the drawing of an operating cooling water system (see figure below).

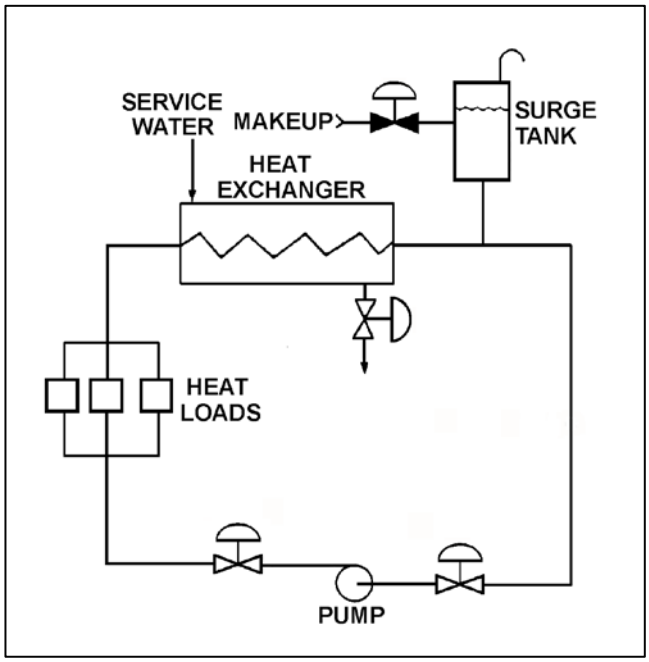
How will the centrifugal pump flow rate be affected if the surge tank level decreases from 8 feet to 4 feet? (Assume the pump maintains adequate net positive suction head.)

A. Pump flow rate will increase.

B. Pump flow rate will decrease.

C. Pump flow rate will remain the same.

D. Pump flow rate will oscillate.



QUESTION: 13

Use the following drawing of system and pump operating curves for a positive displacement pump with discharge relief valve protection to answer the following question.

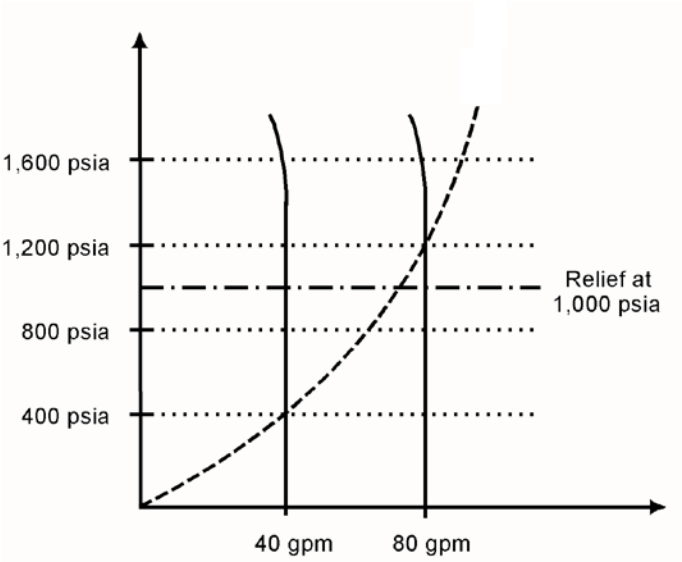
A positive displacement pump is initially supplying water at 40 gpm with a pump discharge pressure of 400 psia. If pump speed is increased until pump flow rate is 80 gpm, what is the new pump discharge pressure?

A. 800 psia

B. 1,000 psia

C. 1,200 psia

D. 1,600 psia



QUESTION: 14

Initially, an AC induction motor is operating with the following steady-state conditions:

Motor current	= 25 amps
Average stator winding temperature	= 140°F
Ambient temperature	= 90°F

Assume the stator winding electrical resistance, motor heat transfer properties, and ambient temperature do not change. If a change in motor load causes the motor current to increase to 50 amps, which one of the following will be the new steady-state average stator winding temperature?

A. 190°F

B. 200°F

C. 280°F

D. 290°F

QUESTION: 15

To minimize the duration of high starting current, an AC induction motor should be started _____ to _____ the stator counter electromotive force.

A. unloaded; quickly establish

B. unloaded; delay

C. partially loaded; quickly establish

D. partially loaded; delay

QUESTION: 16

The rate of heat transfer between two liquids in a single-phase heat exchanger will decrease if the... (Assume constant specific heat capacities.)

A. inlet temperatures of both liquids decrease by 20°F.

B. inlet temperatures of both liquids increase by 20°F.

C. flow rate of the colder liquid decreases by 10 percent.

D. flow rate of the hotter liquid increases by 10 percent.

QUESTION: 17

A reactor is shut down with the residual heat removal (RHR) system in service. Assume that only the RHR heat exchangers are removing heat from the reactor coolant system (RCS), and the RHR system provides complete thermal mixing of the RCS. Also, assume that core decay heat is the only source of heat addition to the RCS.

Given the following current information:

Reactor core rated thermal power	=	2,950 MW
Core decay heat rate	=	0.6 percent of rated thermal power
RHR system heat removal rate	=	4.7×10^7 Btu/hr
RHR and reactor coolant c_p	=	1.05 Btu/lbm-°F
Combined RCS and RHR coolant mass	=	450,000 lbm

Which one of the following actions will establish a reactor coolant heatup rate between 10°F/hour and 20°F/hour?

A. Increase RHR heat exchanger flow rate to reduce the heatup rate by 10°F/hour.

B. Increase RHR heat exchanger flow rate to reduce the heatup rate by 110°F/hour.

C. Decrease RHR heat exchanger flow rate to increase the heatup rate by 10°F/hour.

D. Decrease RHR heat exchanger flow rate to increase the heatup rate by 110°F/hour.

QUESTION: 18

A sudden increase in the conductivity of water at the outlet of a demineralizer may result from...

A. increased demineralizer flow rate.

B. reduced demineralizer inlet temperature.

C. increased demineralizer effluent pressure.

D. reduced demineralizer inlet conductivity.

QUESTION: 19

A mixed-bed ion exchanger is being used to process reactor coolant letdown. The ion exchanger is boron-saturated for the existing reactor coolant conditions. Which one of the following describes a system change and resulting effect that will cause the boron concentration in the ion exchanger outlet water to be greater than the boron concentration in the inlet water?

A. An increase in reactor coolant ionic impurities with higher relative affinities for the resin exchange sites will displace borate ions from the resin exchange sites.

B. An increase in reactor coolant suspended solids with greater mass than the borate ions will mechanically remove borate ions from the resin exchange sites.

C. A decrease in the temperature of the inlet water will lower the relative affinity of the resin for the borate ions, which releases borate ions from the resin exchange sites.

D. A decrease in the flow rate through the ion exchanger will lower the retention capacity of the resin, which releases borate ions from the resin exchange sites.

QUESTION: 20

Refer to the drawing of a valve motor control circuit (see figure below) for a valve that is currently fully closed and has a 10-second stroke time.

Note: Limit switch (LS) contacts are shown open regardless of valve position, but relay contacts are shown open/closed according to the standard convention for control circuit drawings.

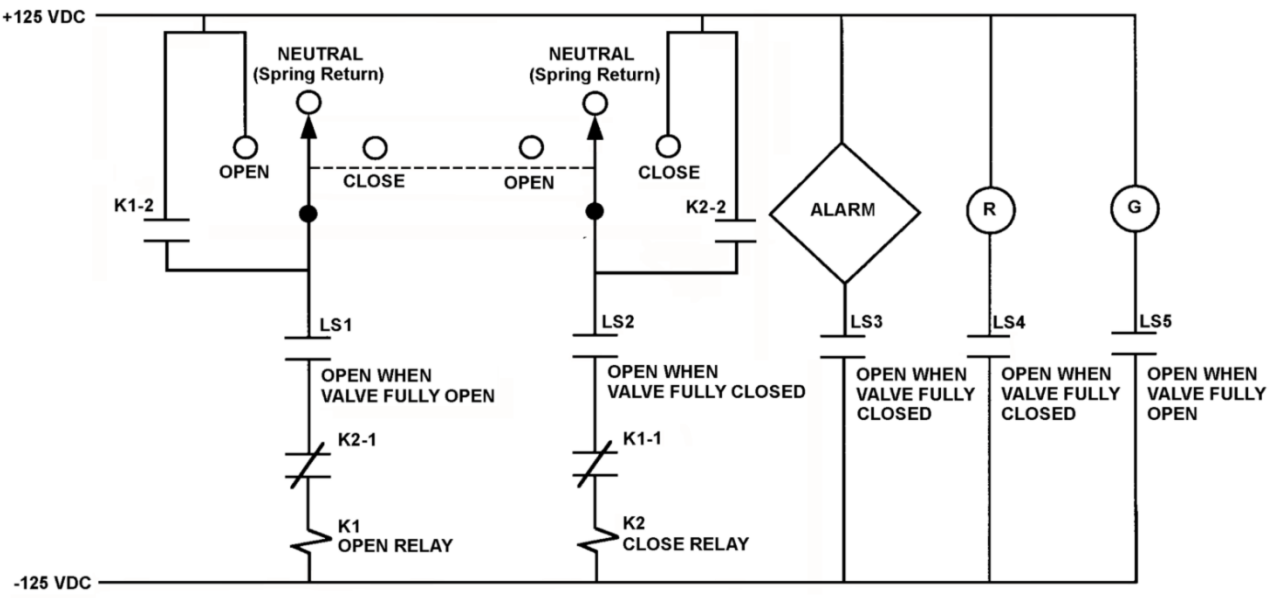
The operator takes the control switch to OPEN. Two seconds later, after verifying the valve is opening, the operator releases the control switch. Which one of the following describes the alarm response after the control switch is released?

A. The alarm will activate after approximately 8 seconds.

B. The alarm will <u>not</u> activate until additional operator action is taken.

C. The alarm will remain activated for approximately 8 seconds, and then deactivate.

D. The alarm will remain activated until additional operator action is taken.



VALVE MOTOR CONTROL CIRCUIT

QUESTION: 21

Under which one of the following pre-existing conditions will closing a breaker between two electrical generators cause a sudden large and possibly damaging mechanical torque to be exerted on both of the generators?

A. One generator is supplying a 3 percent higher voltage than the other.

B. One generator is supplying a 3 percent higher frequency than the other.

C. The voltage of one generator is out of phase with the other by 30E.

D. The capacity of one generator is twice that of the other generator.

QUESTION: 22

While remotely investigating the condition of a normally-open 480 VAC motor control center (MCC) feeder breaker, an operator observes the following indications:

Green breaker position indicating light is out.
Red breaker position indicating light is lit.
MCC voltmeter indicates 480 VAC.
MCC ammeter indicates zero amperes.

Based on these indications, the operator should report that the feeder breaker is _____ and racked _____.

A. open; in

B. closed; in

C. open; to the TEST position

D. closed; to the TEST position

QUESTION: 23

Which one of the following types of neutrons in a reactor is more likely to cause fission of a U-238 nucleus in the reactor fuel? (Assume that each type of neutron remains in the reactor until it interacts with a U-238 nucleus.)

A. A thermal neutron.

B. A prompt fission neutron beginning to slow down.

C. A delayed fission neutron beginning to slow down.

D. A fission neutron at a U-238 resonance energy.

QUESTION: 24

Initially, a nuclear power plant is operating at steady-state 70 percent power near the middle of a fuel cycle when a control rod drops into the core. The reactor does not trip. Consider the following two possible operator responses:

Response 1: An operator adjusts the reactor coolant system (RCS) boron concentration to restore the initial RCS temperatures.

Response 2: An operator partially withdraws some of the remaining control rods to restore the initial RCS temperatures.

In a comparison between the two responses, which response, if any, will result in the smaller available shutdown margin (SDM) when the plant is stabilized at 70 percent power, and why?

A. Response 1, because a smaller (than response 2) amount of negative reactivity will be added by the control rods upon a reactor trip.

B. Response 2, because a greater (than response 1) amount of positive reactivity will be added by the RCS cooldown that occurs immediately after a reactor trip.

C. Both responses will produce the same available SDM, because both responses will stabilize the plant at the same initial steady-state power level and RCS temperatures.

D. Both responses will produce the same available SDM, because both responses will add the same amount of positive reactivity to compensate for the dropped control rod.

QUESTION: 25

A nuclear power plant has been operating at 100 percent power for two months when a reactor trip occurs. Two months after the reactor trip, with all control rods still fully inserted, a stable count rate of 20 cps is indicated on the source range nuclear instruments.

The majority of the source range detector output is being caused by the interaction of _____ with the detector.

A. intrinsic source neutrons

B. fission gammas from previous power operation

C. fission neutrons from subcritical multiplication

D. delayed fission neutrons from previous power operation

QUESTION: 26

The amount of pure water required to decrease the reactor coolant boron concentration by 20 ppm at 100 ppm is approximately _____ the amount of pure water required to decrease the reactor coolant boron concentration by 20 ppm at 1,000 ppm.

A. one-tenth

B. the same as

C. 10 times

D. 100 times

QUESTION: 27

Ignoring the effects of changes in fission product poisons, which one of the following power changes requires the greatest amount of positive reactivity addition?

A. 3 percent to 5 percent

B. 5 percent to 15 percent

C. 15 percent to 30 percent

D. 30 percent to 60 percent

QUESTION: 28

The main reason for designing and operating a reactor with a flattened neutron flux distribution is to...

A. provide even burnup of control rods.

B. reduce neutron leakage from the core.

C. achieve a higher average power density.

D. provide more accurate nuclear power indication.

QUESTION: 29

A reactor has been operating at 100 percent power for several weeks near the middle of a fuel cycle with all control rods fully withdrawn. Which one of the following describes why most of the power is being produced in the lower half of the reactor core?

A. Xenon-135 concentration is lower in the lower half of the core.

B. The moderator to fuel ratio is lower in the lower half of the core.

C. The fuel loading in the lower half of the core contains a higher uranium-235 enrichment.

D. The moderator temperature coefficient of reactivity is adding less negative reactivity in the lower half of the core.

QUESTION: 30

Which one of the following has the greatest microscopic cross section for absorption of a thermal neutron?

A. Uranium-235

B. Boron-10

C. Samarium-149

D. Xenon-135

QUESTION: 31

A reactor was operating at 100 percent power for 8 weeks when a reactor trip occurred. The reactor was critical 6 hours later and power was increased to 100 percent over the next 6 hours.

What was the status of xenon-135 concentration when power reached 100 percent?

A. Increasing toward an equilibrium value.

B. Burning out faster than it is being produced.

C. Increasing toward a peak value.

D. At equilibrium.

QUESTION: 32

Refer to the graph of critical boron concentration versus core burnup for a reactor following a refueling outage (See figure below.).

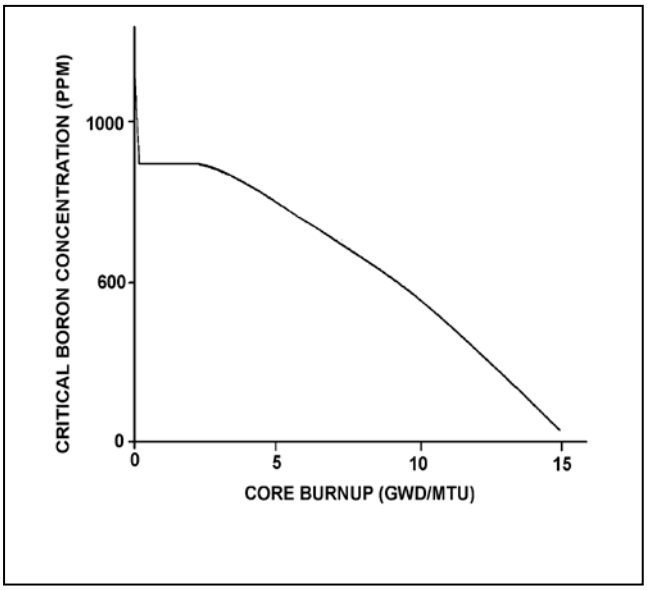
Which one of the following is primarily responsible for the shape of the curve from the middle of core life to the end of core life?

A. Fuel depletion

B. Fission product buildup

C. Burnable poison burnout

D. Conversion of U-238 to Pu-239



QUESTION: 33

An estimated critical rod position (ECP) has been calculated for criticality to occur 15 hours after a reactor trip from long-term 100 percent power operation. Which one of the following conditions would cause the actual critical rod position to be higher than the ECP?

A. A 90 percent value for reactor power was used for power defect determination in the ECP calculation.

B. Reactor criticality is achieved approximately 2 hours earlier than anticipated.

C. Steam generator pressures are decreased by 100 psi just prior to criticality.

D. Current boron concentration is 10 ppm lower than the value used in the ECP calculation.

QUESTION: 34

A reactor and plant startup is in progress. Reactor power is currently 5.0×10^{-5} percent and increasing, with a constant startup rate of 0.2 DPM. Reactivity is not changing.

The reactor is currently _____, at a power level that is _____ the point of adding heat.

A. critical; less than

B. critical; greater than

C. supercritical; less than

D. supercritical; greater than

QUESTION: 35

A nuclear power plant is operating at steady-state 90 percent power near the end of a fuel cycle with manual rod control when a turbine control system malfunction opens the main turbine steam inlet valves an additional 5 percent. Reactor power will initially...

A. increase, because the rate of neutron absorption in the moderator initially decreases.

B. increase, because the rate of neutron absorption at U-238 resonant energies initially decreases.

C. decrease, because the rate of neutron absorption in the moderator initially increases.

D. decrease, because the rate of neutron absorption at U-238 resonant energies initially increases.

QUESTION: 36

Which one of the following is the reason for inserting control rods in a predetermined sequence during a normal reactor shutdown?

A. To prevent uneven fuel burnup.

B. To prevent an excessive reactor coolant system cooldown rate.

C. To prevent abnormally high local power peaks.

D. To prevent divergent xenon-135 oscillations.

QUESTION: 37

Refer to the drawing of two water storage tanks with four differential pressure (D/P) level detectors (see figure below).

The tanks are identical and are being maintained at the same constant water level with 17 psia gas pressure above the water. The tanks are surrounded by standard atmospheric pressure. The temperature of the water in the tanks and reference legs is 70°F.

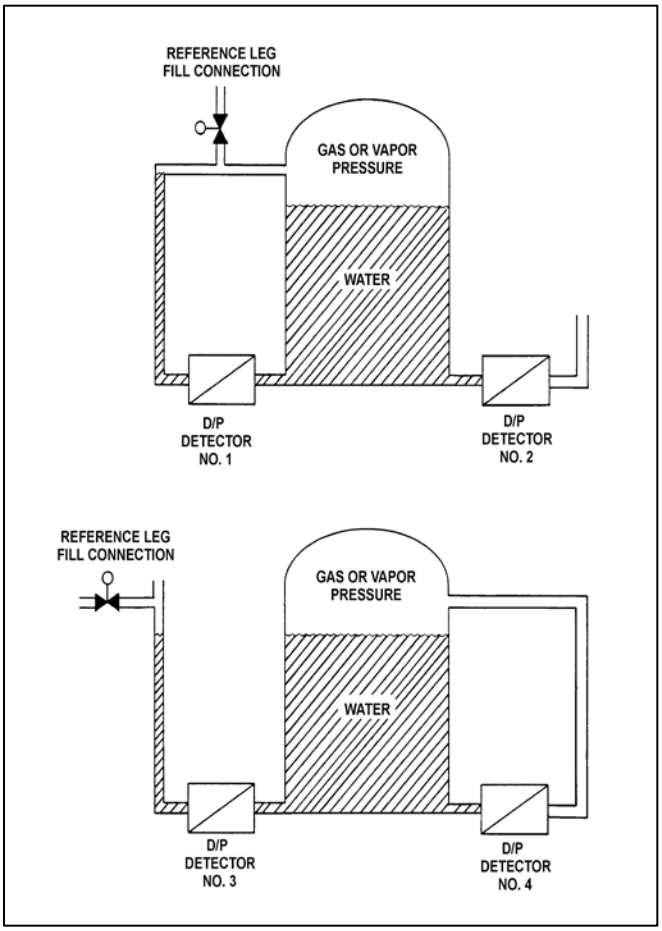
Which one of the level detectors is sensing the greatest D/P?

A. No. 1

B. No. 2

C. No. 3

D. No. 4



QUESTION: 38

Refer to the drawing of a section of 6-inch diameter pipe containing subcooled water flowing from left to right at 100 gpm (see figure below). The pipe is frictionless and no heat transfer is occurring. Point B is 10 feet higher in elevation than point A.

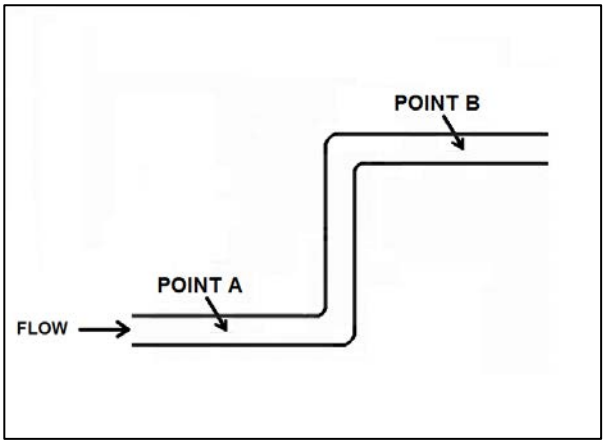
How does the enthalpy of the water at point A compare to point B?

A. The enthalpy of the water at point A is smaller, because some of the water's kinetic energy is converted to enthalpy as it flows to point B.

B. The enthalpy of the water at point A is greater, because some of the water's enthalpy is converted to potential energy as it flows to point B.

C. The enthalpy of the water at points A and B is the same, because the pipe is frictionless and no heat transfer is occurring.

D. The enthalpy of the water at points A and B is the same, because the total energy of the water does not change from point A to point B.



QUESTION: 39

A reactor is shut down with reactor coolant system (RCS) pressure at 1,000 psia and core decay heat is being removed via the steam generators (SGs). What pressure must be maintained in the SGs to obtain a 50°F subcooling margin in the reactor coolant leaving the SGs? (Assume the reactor coolant leaves the SGs at the SG saturation temperature.)

A. 550 psia

B. 600 psia

C. 650 psia

D. 700 psia

QUESTION: 40

The pressure of a saturated steam-water mixture is 760 psia.

Which one of the following parameter values, when paired with the pressure of the mixture, provides insufficient information to determine the specific volume of the mixture?

A. Quality is 84.6 percent.

B. Temperature is 512.4°F.

C. Enthalpy is 764.5 Btu/lbm.

D. Entropy is 0.88 Btu/lbm-ER.

QUESTION: 41

Refer to the drawing of two 1,000 ft^3 pressure vessels with installed relief valves (see figure below).

Both vessels are in saturated conditions at 281°F and approximately 35 psig. Vessel A is completely filled with saturated water. Vessel B contains one-half saturated steam (100 percent quality) and one-half saturated water (0 percent quality) by volume. Both vessels are protected by identical relief valves.

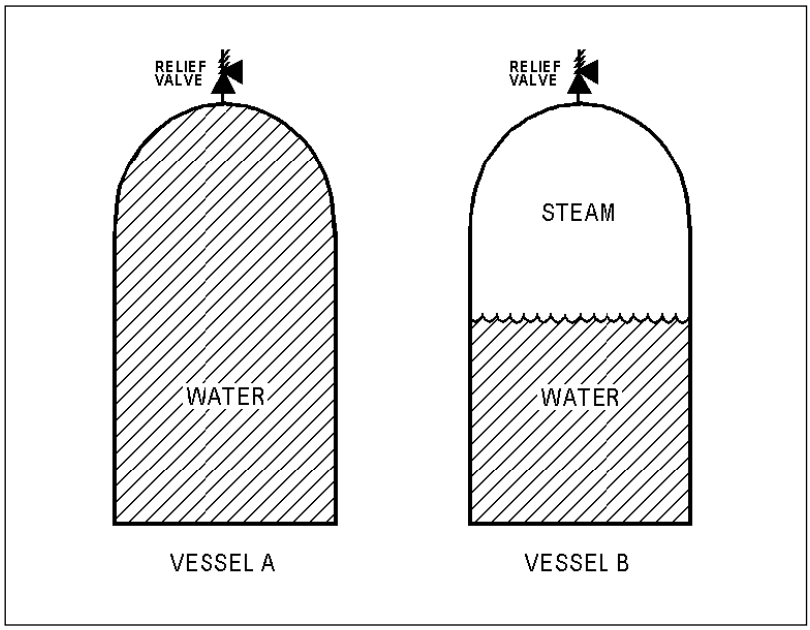
If both relief valves begin to leak at a rate of 0.1 percent of design flow, the higher temperature fluid will initially be leaving the relief valve of vessel _____. And, if 100 lbm of fluid is released through both relief valves, the larger pressure decrease will occur in vessel _____.

A. A; A

B. A; B

C. B; A

D. B; B



QUESTION: 42

In a nuclear power plant main turbine, if the moisture content of the inlet steam increases from 0.25 percent to 0.5 percent at the same pressure, the main turbine work output will…

A. increase due to the greater initial enthalpy of the inlet steam.

B. increase due to the increased momentum transfer from water droplets impacting the turbine blading.

C. decrease due to the lower initial temperature of the inlet steam.

D. decrease due to the increased braking action from water droplets impacting the turbine blading.

QUESTION: 43

Refer to the drawing of two lengths of 16-inch diameter pipe, each containing an identical automatic isolation valve. The actual pipe lengths are proportional to their symbols in the drawing.

Water is flowing at 10,000 gpm through each pipe when both isolation valves instantly close.

Consider two cases:

Case 1: The water temperature upstream of both valves is 65°F.
Case 2: The water temperature is 85°F upstream of valve A, and 65°F upstream of valve B.

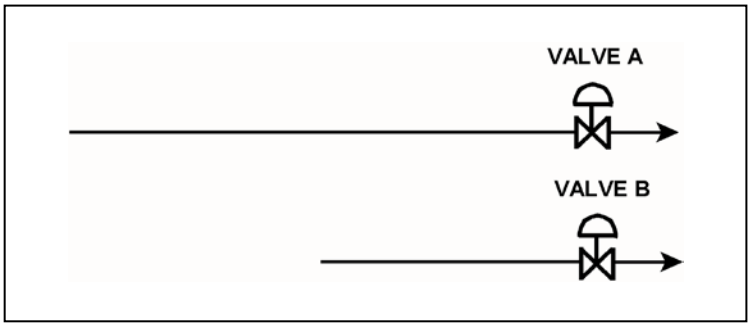
For which case(s), if any, will valve A experience a pressure spike that is greater than the pressure spike at valve B?

A. Case 1 only

B. Case 2 only

C. Both cases

D. Neither case



QUESTION: 44

The volumetric flow rate of cooling water entering a heat exchanger is 500 gpm.

Given the following:

- Cooling water pressure entering and leaving the heat exchanger is 10 psig.
- Cooling water inlet temperature is 90°F.
- Cooling water outlet temperature is 160°F.
- Heat exchanger inlet and outlet piping have the same diameter.

What is the approximate volumetric flow rate of the cooling water exiting the heat exchanger?

A. 496 gpm

B. 500 gpm

C. 504 gpm

D. 509 gpm

QUESTION: 45

The power range nuclear instruments have been adjusted to 100 percent based on a calculated heat balance. Which one of the following will result in indicated reactor power being lower than actual reactor power?

A. The feedwater temperature used in the heat balance calculation was 20°F higher than actual feedwater temperature.

B. The reactor coolant pump heat input value used in the heat balance was 10 percent lower than actual reactor coolant pump heat input.

C. The feedwater flow rate used in the heat balance calculation was 10 percent higher than actual feedwater flow rate.

D. The operator miscalculated the enthalpy of the steam exiting the steam generators to be 10 Btu/lbm higher than actual.

QUESTION: 46

Case 1: Subcooled reactor coolant enters the bottom of a fuel assembly in a reactor operating at power. As the coolant flows upward through the fuel assembly, the water heats up and exits the fuel assembly still subcooled.

Case 2: Same as above except that reactor pressure is decreased such that the coolant begins to boil halfway up the fuel assembly, which results in a saturated steam-water mixture exiting the fuel assembly.

Assume that departure from nucleate boiling is avoided in both cases and that power level does not change. As compared to Case 1, the average fuel temperature for Case 2 will be _____ because boiling is a _____ efficient method of heat transfer.

A. higher; more

B. higher; less

C. lower; more

D. lower; less

QUESTION: 47

A reactor is producing 3,400 MW of thermal output with a reactor vessel differential temperature (ΔT) of 60°F and a reactor vessel mass flow rate of 1.0×10^8 lbm/hr. If core ΔT is 63.6°F, what is core bypass mass flow rate? (Assume bypass flow ΔT equals 0°F.)

A. 5.66×10^6 lbm/hr

B. 8.40×10^6 lbm/hr

C. 3.60×10^7 lbm/hr

D. 9.43×10^7 lbm/hr

QUESTION: 48

Which one of the following conditions must occur to sustain natural convection in a fluid system?

A. Subcooling of the fluid.

B. A phase change in the fluid.

C. A density change in the fluid.

D. Radiative heat transfer to the fluid.

QUESTION: 49

A nuclear power plant is operating at steady-state 80 percent power in the middle of a fuel cycle. All control rods are fully withdrawn and in manual control. Core axial power distribution is peaked below the core midplane.

Which one of the following will cause the maximum axial peaking (or hot channel) factor to initially decrease?

A. One bank of control rods is inserted 10 percent.

B. Turbine load/reactor power is reduced by 10 percent.

C. Reactor coolant system boron concentration is reduced by 10 ppm.

D. A control rod located at the edge of the core fully inserts into the core.

QUESTION: 50

A reactor is shut down for refueling. During the shutdown, a reactor vessel metal specimen was removed from the reactor vessel for testing. The specimen was last tested six years ago and then returned to its original location in the reactor vessel. During the subsequent six years, the reactor has completed several 18 month fuel cycles with an average power level of 85 percent.

The tests determined that the nil-ductility transition (NDT) temperature of the specimen has remained unchanged at 44°F since it was last tested. Which one of the following conclusions is warranted?

A. The test results are credible; however, the reactor vessel is more susceptible to brittle fracture now than six years ago.

B. The test results are credible; however, the reactor vessel is less susceptible to brittle fracture now than six years ago.

C. The test results are questionable because the specimen NDT temperature should have increased since it was last tested.

D. The test results are questionable because the specimen NDT temperature should have decreased since it was last tested.

***** FINAL ANSWER KEY *****

MARCH 2018 GENERIC FUNDAMENTALS EXAMINATION
PRESSURIZED WATER REACTOR - ANSWER KEY

FORM A	FORM B	ANS.	FORM A	FORM B	ANS.
1	15	A	26	40	C
2	16	D	27	41	D
3	17	D	28	42	C
4	18	B	29	43	D
5	19	A	30	44	D
6	20	B	31	45	B
7	21	A	32	46	A
8	22	B	33	47	B
9	23	D	34	48	C
10	24	D	35	49	B
11	25	A	36	50	C
12	26	C	37	1	B
13	27	B	38	2	B
14	28	D	39	3	C
15	29	A	40	4	B
16	30	C	41	5	D
17	31	A	42	6	D
18	32	A	43	7	D
19	33	A	44	8	D
20	34	D	45	9	A
21	35	C	46	10	C
22	36	B	47	11	A
23	37	B	48	12	C
24	38	A	49	13	B
25	39	C	50	14	C