

REACTOR COOLANT SYSTEM AND CONNECTED SYSTEMS CHAPTER 5 FIGURES Revision: 25
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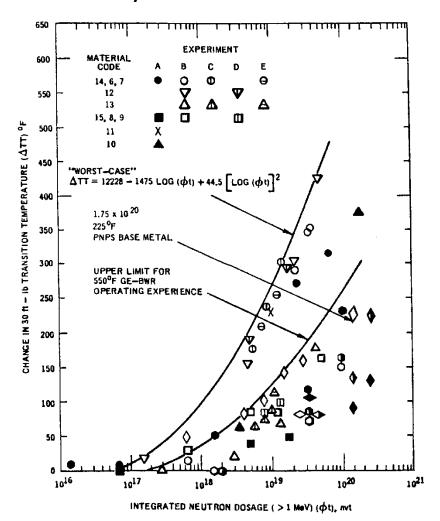
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EFFECT OF IRRADIATION ON VARIOUS HEATS OF A302B/A533B — CLASS 1



GENERAL ELECTRIC SURVEILLANCE PROGRAM TEST RESULTS PLOTTED FOR COMPARISON WITH REFERENCE DATA (2)

- ONPS BASE METAL
- DNPS WELD METAL
- DNPS HAZ
- O BIG ROCK BASE METAL
- BIG ROCK WELD METAL
- BIG ROCK HAZ
- A HUMBOLDT BASE METAL
- → HUMBOLDT WELD METAL
- THUMBOLDT HAZ



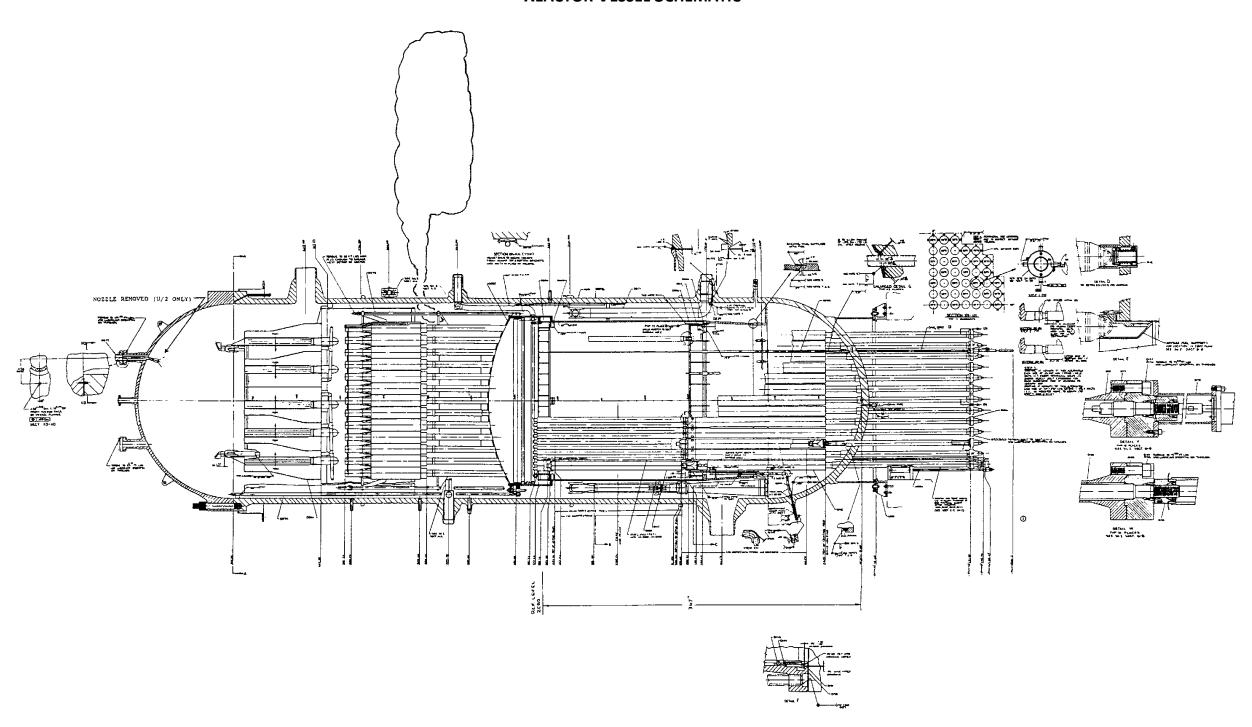
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REACTOR VESSEL SCHEMATIC



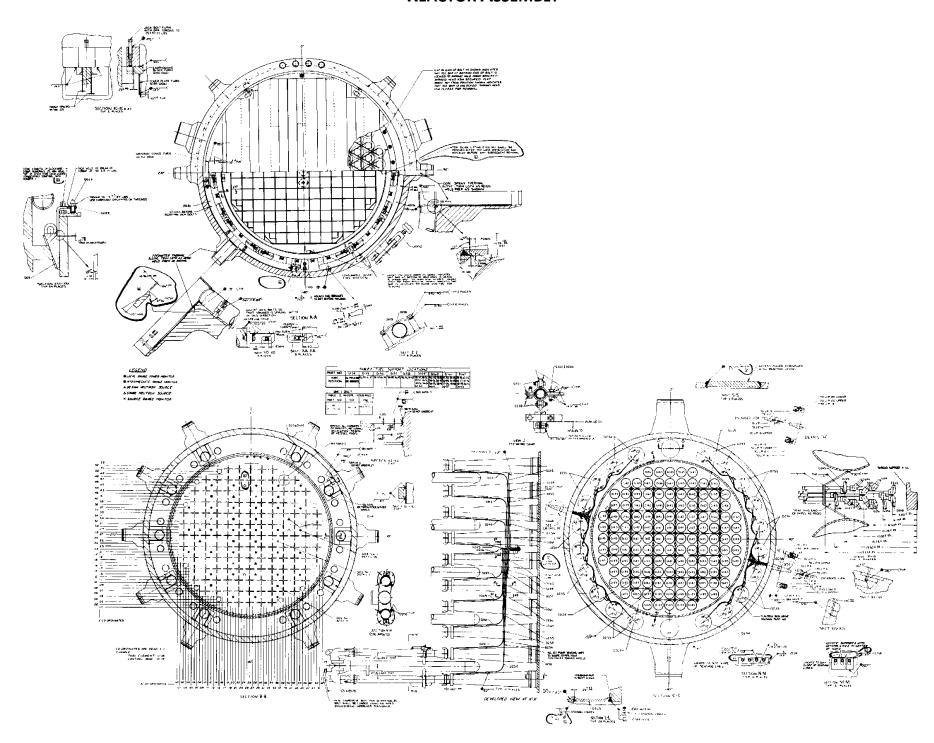


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REACTOR ASSEMBLY





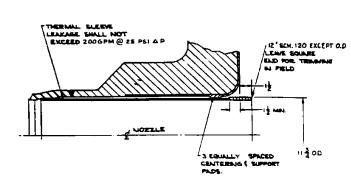
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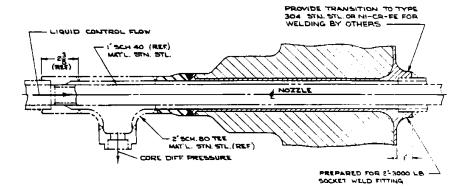
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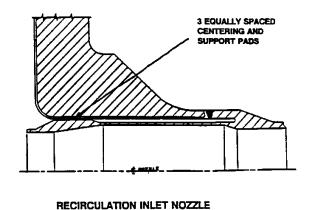
REACTOR VESSEL NOZZLES AND PENETRATIONS



FEEDWATER NOZZLE

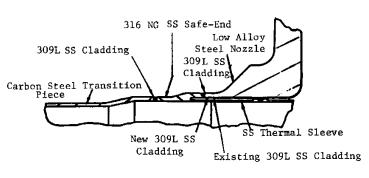


CORE DIFFERENTIAL PRESSURE &

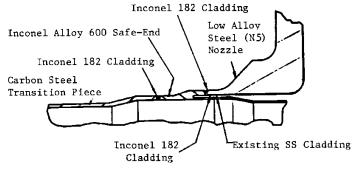


INSTR. NOZZLE ON HEAD

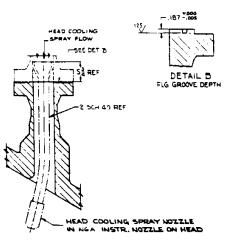
NOTE: THIS DRAWING PROVIDES FUNCTIONAL REQUIREMENTS 4 IS NOT INDICATIVE OF DETA . DESIGN OF NOZZLES OR THERMAL SLEEVES. THE SUPPLIER SHALL PROVIDE DETAIL DESIGN TO CARRY IMPOSED LOADS & ACCOMMODATE THERMAL TRANSIENTS



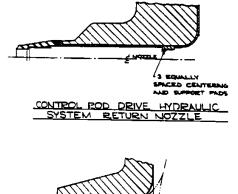
CORE SPRAY NOZZLE - UNIT 1



CORE SPRAY NOZZLE - UNIT 2



RECIRCULATION OUTLET NOZZLE



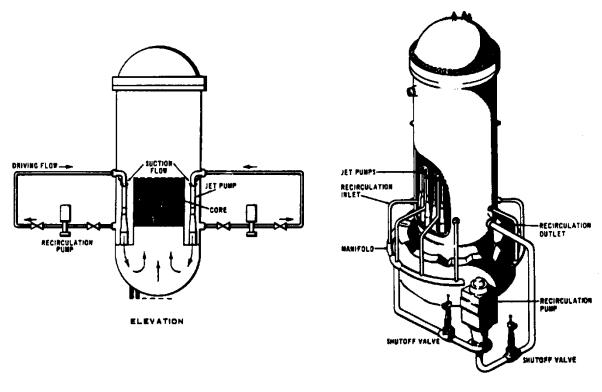


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RECIRCULATION SYSTEM — ELEVATION (ISOMETRIC)



ISOMETRIC

SECURITY-RELATED INFORMATION WITHHOLD UNDER 10 CFR 2.390



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SECU	RITY-RELATED INFORMA	TION WITHHOLD UND	ER 10 CFR 2.390	



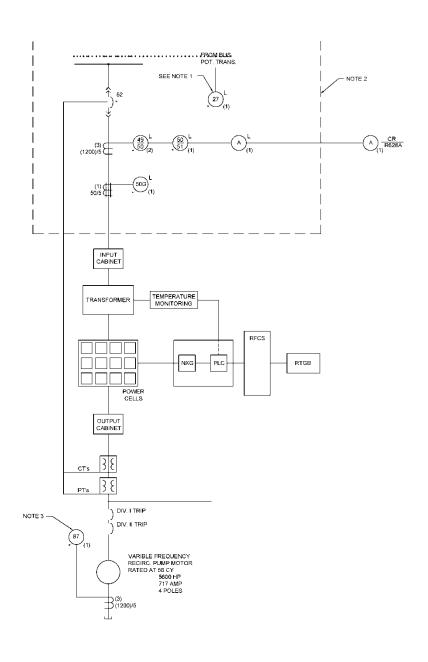
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RECIRCULATION PUMP - ONE LINE METER AND RELAY DIAGRAM





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NOTES

- 1. Reference Document: Reactor Recirculation System FCD D32-1020.
- 2. Drive motor protective relays and metering shall be furnished by others and mounted on the motor switchgear cubicle.
- 3. These devices shall be part of and mounted on Panel No. P003 (protection and Auxiliary Relay Cubicle).



REACTOR COOLANT SYSTEM AND CONNECTED SYSTEMS CHAPTER 5 FIGURES

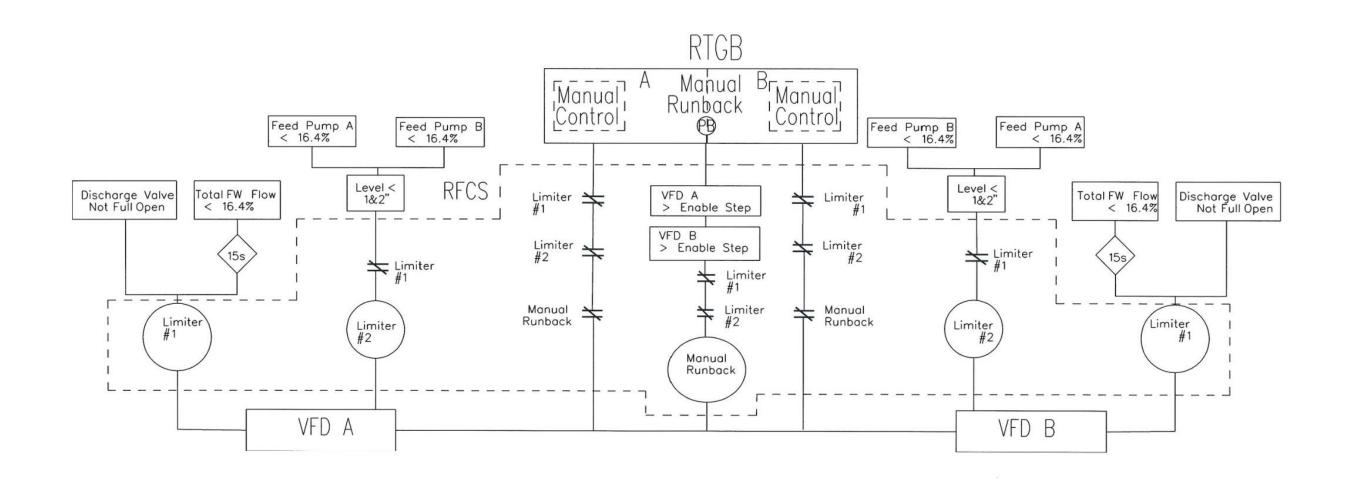
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RECIRCULATION PUMP SPEED CONTROL BLOCK DIAGRAM



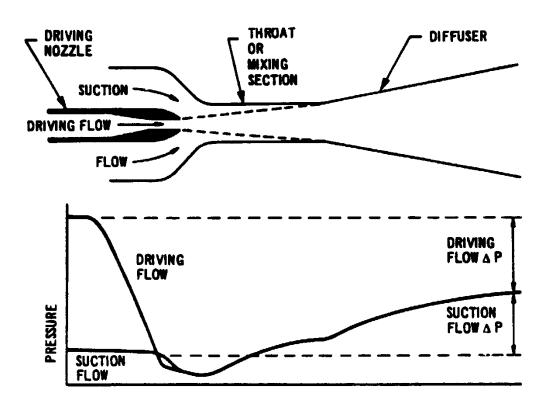


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JET PUMP OPERATING PRINCIPLE





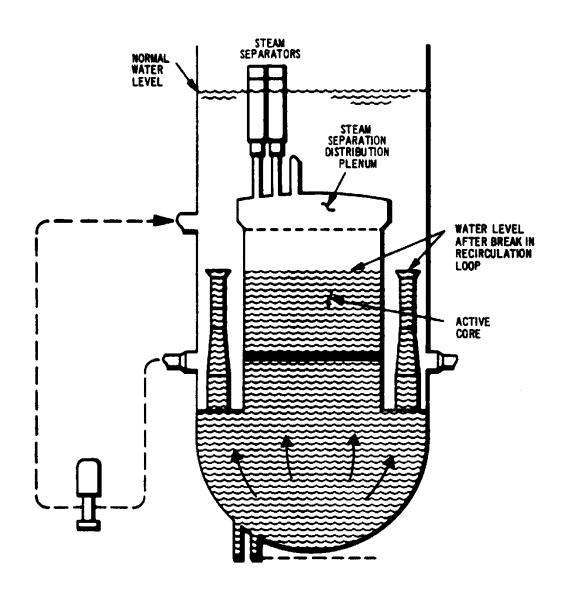
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RECIRCULATION SYSTEM CORE FLOODING CAPABILITY



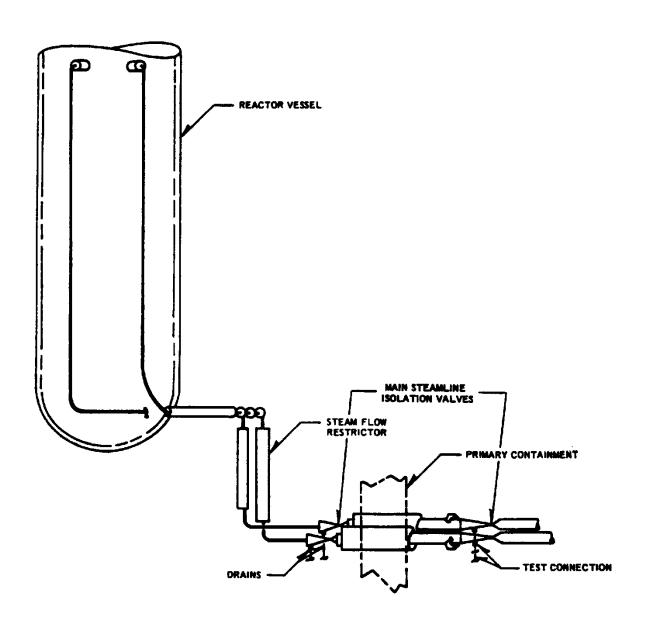


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MAIN STEAM LINE FLOW RESTRICTOR LOCATION



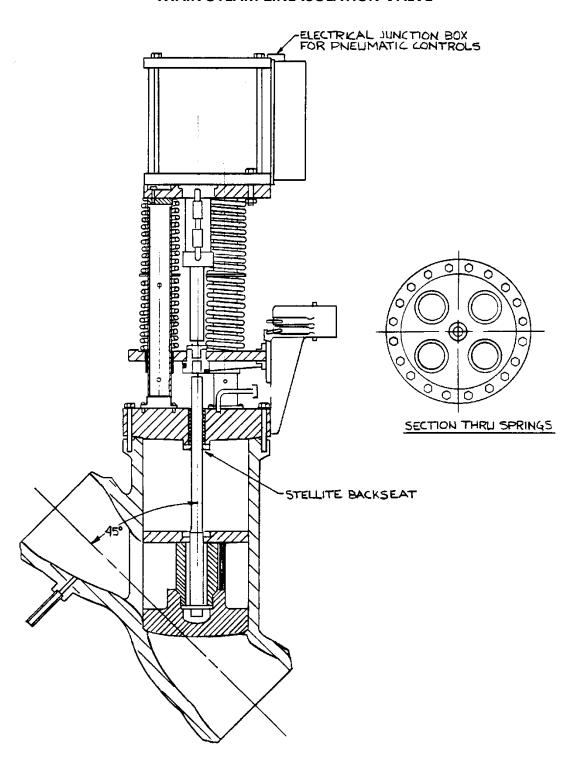


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MAIN STEAM LINE ISOLATION VALVE





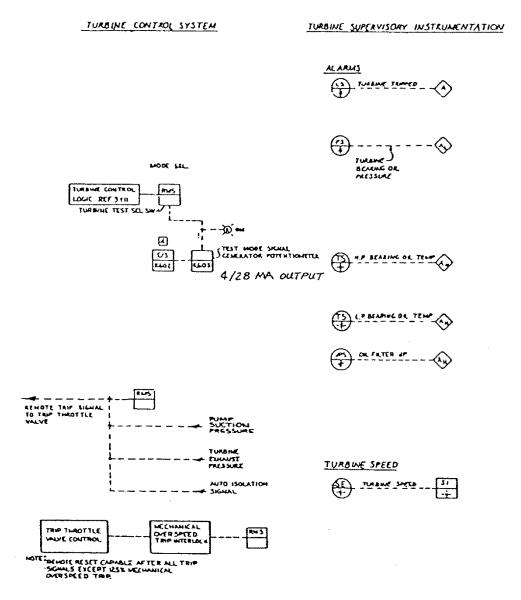
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REACTOR CORE ISOLATION COOLING PUMP TURBINE DETAILS FLOW DIAGRAM



1. For RCIC leak detection arrangement, see Figures 7-19 and 7-20.

Ref. Drawing 9527-D-2529, Rev. 17.



REACTOR COOLANT SYSTEM AND CONNECTED SYSTEMS CHAPTER 5 FIGURES

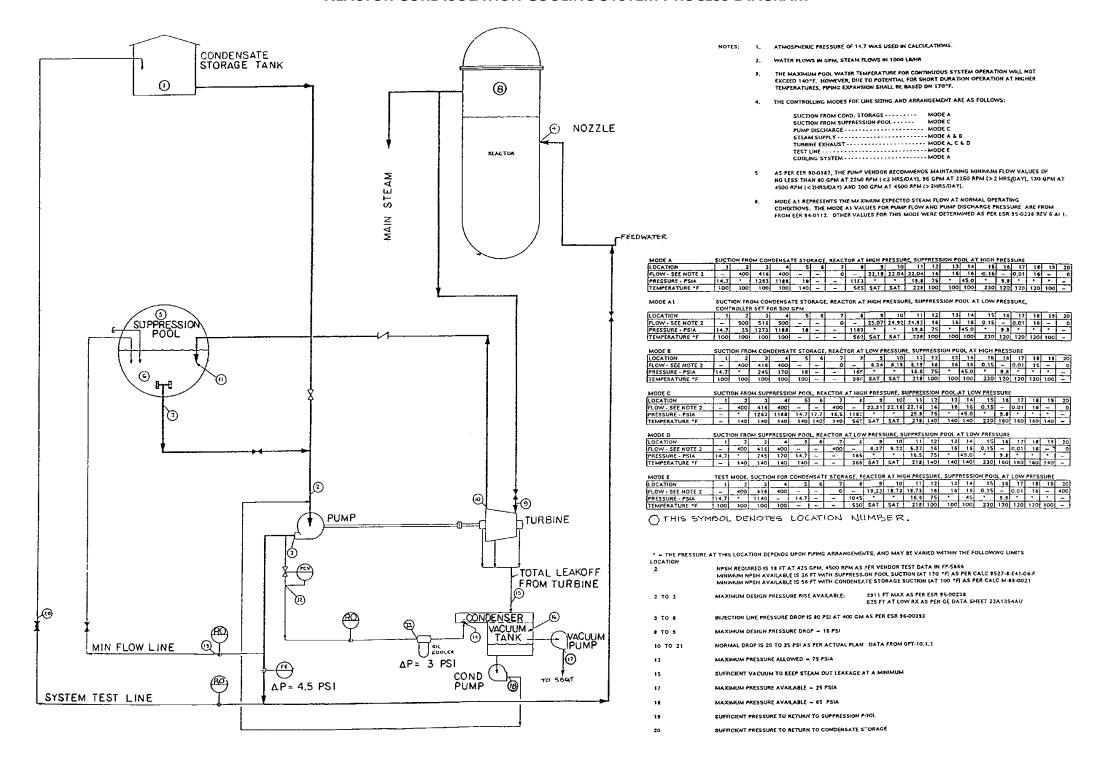
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REACTOR CORE ISOLATION COOLING SYSTEM PROCESS DIAGRAM





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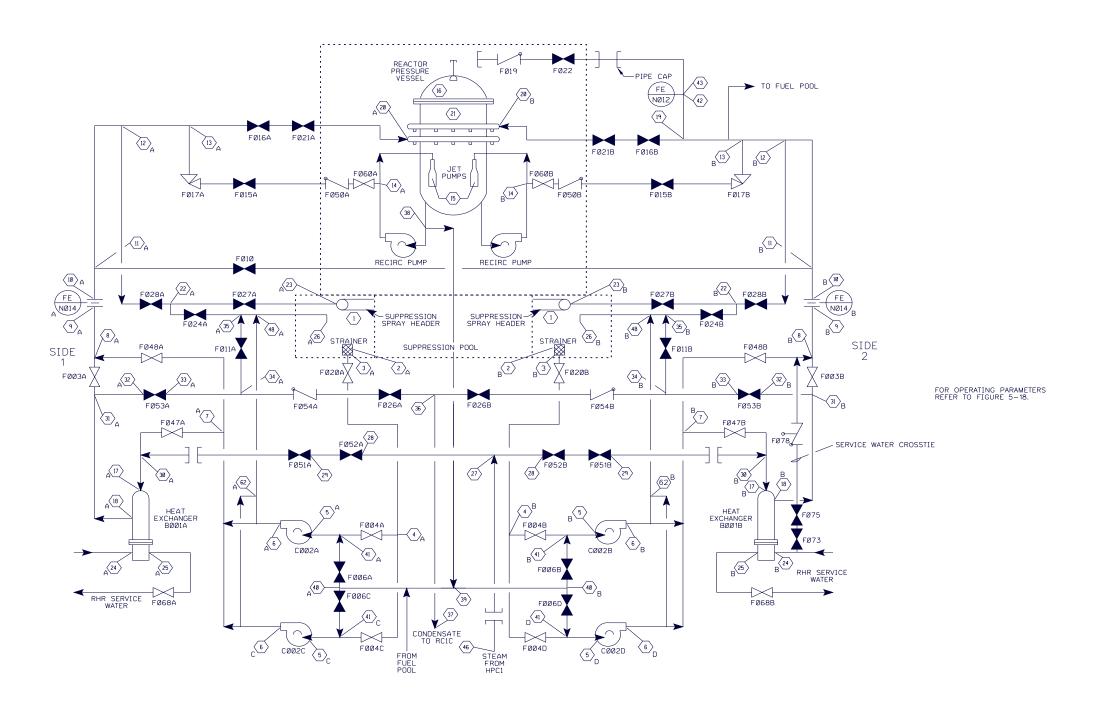
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RESIDUAL HEAT REMOVAL SYSTEM (LPCI) PROCESS DIAGRAM





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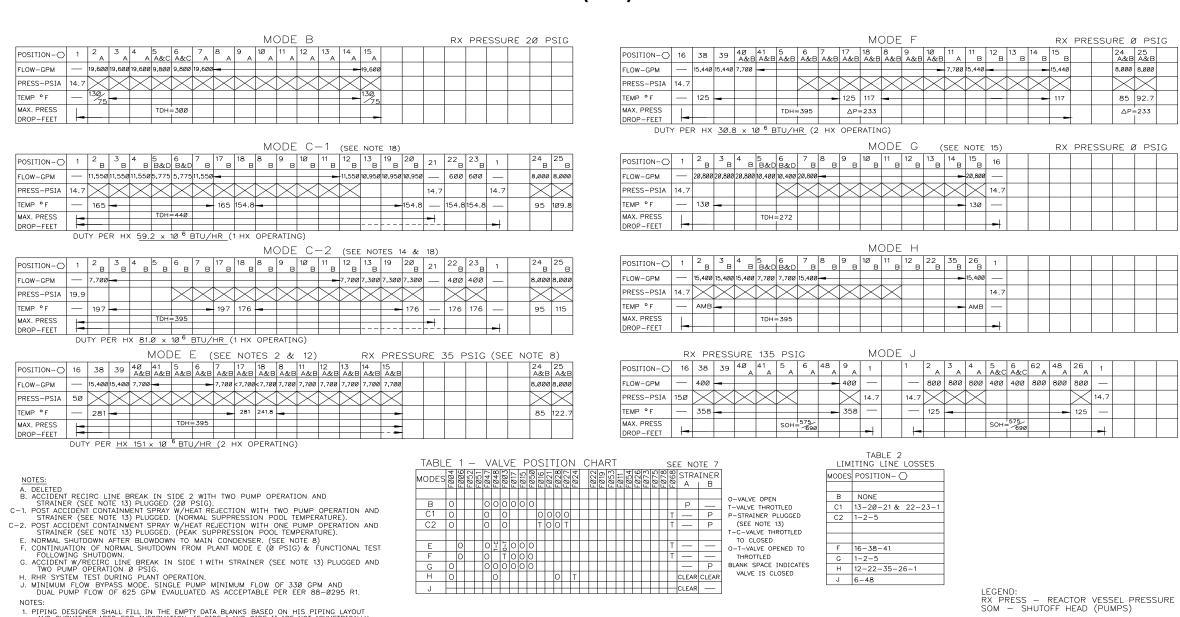
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RESIDUAL HEAT REMOVAL SYSTEM (LPCI) PROCESS DIAGRAM



GREATER THAN 20.2 FEET.

GREATER THAN 20.2 FEET.

16. DRAWING Ø-FP-Ø5555 IS THE PARENT DRAWING TO THIS FIGURE.

17. SEE TABLE 6-9 FOR LONG TERM PRIMARY CONTAINMENT RESPONSE FOR VARIOUS RHR
COOLING MODES.

18. HEAT EXCHANGER DUTY AND TEMPERATURE VALUES OBTAINED FROM GE RHR Hx SPECIFICATION
(21A9227) AND PERFEX VENDOR MANUAL FP-Ø5652.

7. TABLE 1 INDICATES VALVE POSITIONS DURING VARIOUS MODES OF OPERATION.
8. SHUTDOWN OPERATION MAY BE INITIATED AT MAXIMUM REACTOR VESSEL PRESSURE OF 13Ø PSIG.
10. THE PRESSURE SHOWN AT POSITION 1 ON MODE C-2 IS A MIN. VALUE AT PEAK POOL
TEMPERATURE FOR NPSH CALCULATION.
11. THE WEIGHT OF WATER IN THE SHUTDOWN COOLING SUBSYSTEM PIPING, INCLUDING
THE HEAT EXCHANGES AND PUMPS, SHALL NOT EXCEED 225,000 LBS AT 70° F TO PREVENT
DILUTION OF STANDBY LIQUID CONTROL NEUTRON ABSORBER BELOW MINIMUM REQUIREMENTS.

1. PIPING DESIGNER SHALL FILL IN THE EMPTY DATA BLANKS BASED ON HIS PIPING LAYOUT AND SUBMIT TO APED FOR INFORMATION. IF SIDE I AND SIDE II ARE NOT ASYMETRICALLY ARRANCED, VALUES FOR BOTH SHALL BE SUBMITTED, SEE SHEET 2.

2. THE MINIMUM PUMP NPSH AVAILABLE THAT OCCUR DURING MODE E MUST BE EQUAL TO OR GREATER THAN 24 FEET.

GREATER THAN 24 FEET.

3. LELVATIONS ARE NOT INCLUDED IN P VALUES GIVEN ELEVATIONS SHALL BE INCLUDED WHEN DETERMINING FINAL VALUES FOR THE EMPTY DATA BANKS.

4. DASHED LINES INDICATE FLOW DOES NOT PASS THROUGH THESE POINTS.

5. SERVICE WATER CROSS—TIE SHALL BE SIZED TO FLOW 3200 GPM.

6. LINE SHALL BE SIZED BASED ON FLOW SHOWN ON FUEL POOL PROCESS DIAGRAM.

12. 19, 42, 43 AND 44 ARE NO LONGER USED AS FLOWPATH IN MODE E DUE TO REMOVAL OF HEAD SPRAY CAPABILITY.

13. THE PRESSURE DROP ACROSS THE SUCTION STRAINER DURING LOCA MITIGATION IS DETERMINED IN ACCORDANCE WITH REGULATORY GUIDE 1.82 REV. 2 AND THE BWR OWNERS GROUP "UTILITY RESOLUTION GUIDANCE FOR ECCS SUCTION STRAINER REPLACEMENT".

14. THE MINIMUM PUMP NPSH AVAILABLE THAT OCCURS DURING MODE C-2 MUST BE EQUAL TO OR GREATER THAN 16.6 FEET.

15. THE MINIMUM PUMP NPSH AVAILABLE THAT OCCURS DURING MODE G MUST BE EQUAL TO OR CREATER THAN 28.2 FEET.



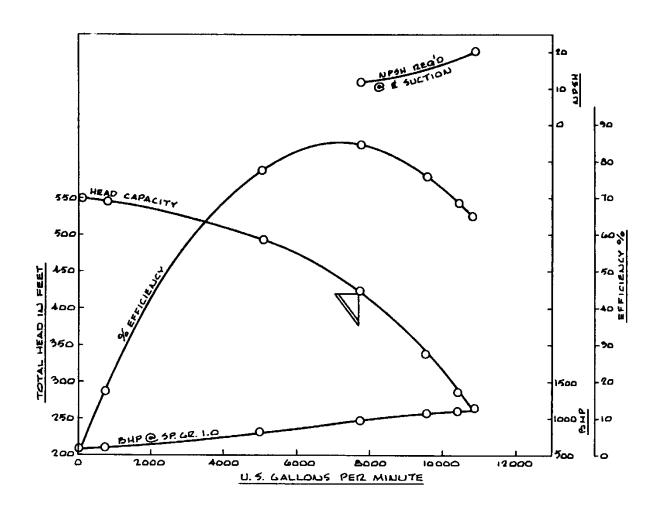
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RESIDUAL HEAT REMOVAL PUMP CHARACTERISTIC CURVE (TYPICAL)





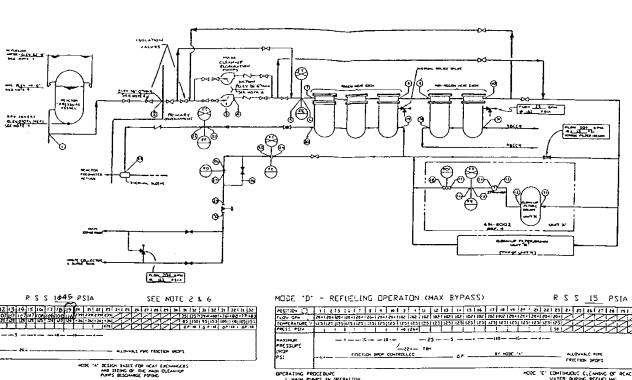
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REACTOR WATER CLEANUP SYSTEM PROCESS DIAGRAM



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REACTOR COOLANT SYSTEM AND CONNECTED SYSTEMS **CHAPTER 5 FIGURES**

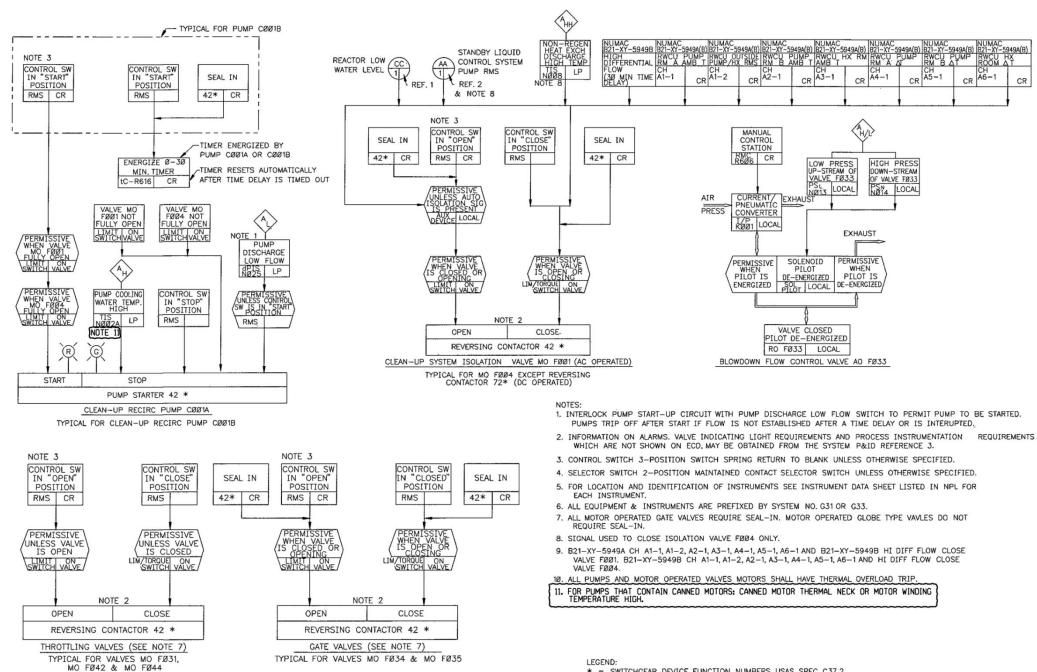
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REACTOR WATER CLEANUP SYSTEM FUNCTIONAL CONTROL DIAGRAM



* = SWITCHGEAR DEVICE FUNCTION NUMBERS USAS SPEC. C37.2.



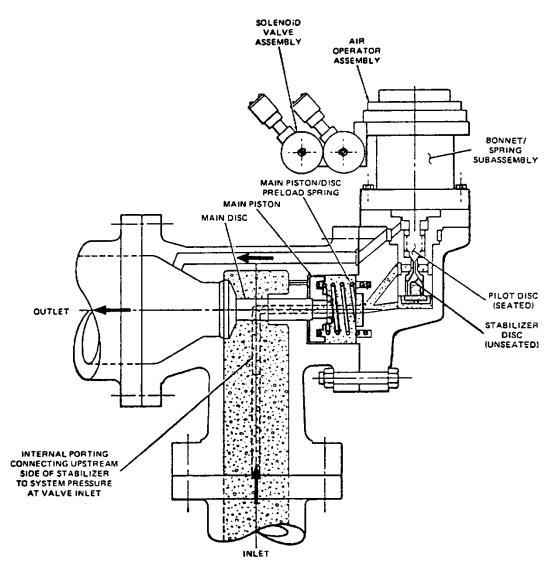
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SAFETY RELIEF VALVE (CLOSED POSITION)



SYSTEM PRESSURE

NOTE: BNP utilizes one solenoid valve, not two as shown.



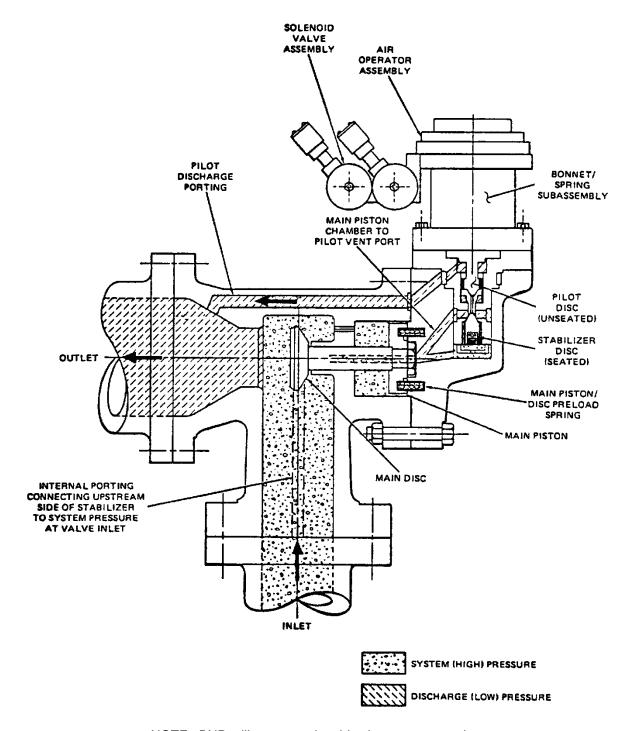
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SAFETY RELIEF VALVE (OPEN POSITION)



NOTE: BNP utilizes one solenoid valve, not two as shown.