ATTACHMENT TO LICENSE AMENDMENT NO. 142

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

<u>REMOVE</u> <u>INSERT</u>

7

Appendix C to Facility Combined License No. NPF-92

<u>REMOVE</u> <u>INSERT</u>

C-294 C-294

(7) Reporting Requirements

- (a) Within 30 days of a change to the initial test program described in UFSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).
- (b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) <u>Incorporation</u>

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 142, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

- (a) Environmental Qualification Program implemented before initial fuel load:
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
 - The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

Table 2.5.2-3 PMS Automatically Actuated Engineered Safety Features

Safeguards Actuation

Containment Isolation

Automatic Depressurization System (ADS) Actuation

Main Feedwater Isolation

Reactor Coolant Pump Trip

CMT Injection

Turbine Trip (Isolated signal to nonsafety equipment)

Steam Line Isolation

Steam Generator Relief Isolation

Steam Generator Blowdown Isolation

Passive Containment Cooling Actuation

Startup Feedwater Isolation

Passive Residual Heat Removal (PRHR) Heat Exchanger Alignment

Block of Boron Dilution

Chemical and Volume Control System (CVS) Makeup Line Isolation

Steam Dump Block (Isolated signal to nonsafety equipment)

Main Control Room Isolation, Air Supply Initiation, and Electrical Load De-energization

Auxiliary Spray and Purification Line and Zinc/Hydrogen Addition Isolation

Containment Air Filtration System Isolation

Normal Residual Heat Removal Isolation

Refueling Cavity and Spent Fuel Pool Cooling System (SFS) Isolation

In-Containment Refueling Water Storage Tank (IRWST) Injection

IRWST Containment Recirculation

CVS Letdown Isolation

Pressurizer Heater Block (Isolated signal to nonsafety equipment)

Containment Vacuum Relief

Component Cooling System Containment Isolation Valve Closure

Table 2.5.2-4 PMS Manually Actuated Functions

Reactor Trip

Safeguards Actuation

Containment Isolation

Depressurization System Stages 1, 2, and 3 Actuation

Depressurization System Stage 4 Actuation

Feedwater Isolation

Core Makeup Tank Injection Actuation

Steam Line Isolation

Passive Containment Cooling Actuation

Passive Residual Heat Removal Heat Exchanger Alignment

IRWST Injection

Containment Recirculation Actuation

Main Control Room Isolation, Air Supply Initiation, and Electrical Load De-energization

Steam Generator Relief Isolation

Chemical and Volume Control System Isolation

Normal Residual Heat Removal System Isolation

Containment Vacuum Relief