

E 05/17/78

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50-250/251

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DOCTYPE: LETTER NOTARIZED: NO

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SUBJECT:

LTR 3 ENCL 0

RESPONSE TO NRC LTR DTD 01/25/78... FURNISHING INFO CONCERNING APPLICANT'S  
DETAILED SCHEDULE FOR PROVIDING AN EVALUATION OF THE EFFECTS OF ASYMMETRIC  
LOSS OF COOLANT ACCIDENT LOADS OF SUBJECT FACILITY.

PLANT NAME: TURKEY PT #3  
TURKEY PT #4

REVIEWER INITIAL: XJM  
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May 9, 1978  
L-78-167

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Office of Nuclear Reactor Regulation  
Attention: Mr. Victor Stello, Jr., Director  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Stello:

Re: Turkey Point Unit Nos. 3 & 4  
Docket Nos. 50-250 and 50-251  
Evaluation of the Overall Asymmetric  
Loss of Coolant Accident Loads  
Operating License Nos. DPR-31 and DPR-41

Florida Power & Light Company was requested by your letter of January 25, 1978, to inform you within ninety (90) days of the detailed schedule for providing an evaluation of the effects of asymmetric loss of coolant accident (LOCA) loads as described in Enclosure 2 of your letter.

Prior to issuance of your letter of January 25, 1978, a task group of utilities was formed, and with the assistance of Westinghouse Electric Corporation, has outlined the steps necessary to complete an evaluation of the type you have requested. The evaluation program prepared by the task group has been broken into three parts, or phases, to facilitate the work effort and to gain some advantage by performing generic analyses applicable to several plants of similar design.

Florida Power & Light Company is presently a member of the task group. We have approved and are participating in Phase A of the program, but have not yet fully reviewed and approved Phases B and C. A summary of the task group program is given below.

Phase A which is essentially complete, consists of a detailed examination of primary loop characteristics for each plant involved in the task group effort to categorize and group the plants for the analyses. Phase B will evaluate the postulated ruptures which have the greatest effect upon the steam generator and reactor coolant pump supports. Phase C will address the reactor vessel response for postulated pipe ruptures at the reactor vessel inlet and outlet nozzles. The need for plant specific analyses will

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A001  
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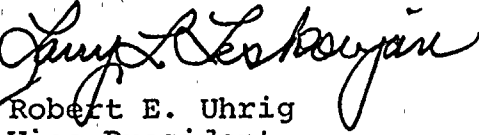
7. 2. 1.

Mr. Victor Stell Jr.  
U. S. Nuclear Regulatory Commission  
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be based on the results of the generic analyses. It is expected that the evaluation will be complete by January, 1980.

Following Florida Power & Light Company's approval of Phases B and C, we will notify the Commission and therein provide a summary of the scope and schedule.

Yours very truly,



for Robert E. Uhrig  
Vice President

REU:WAK:PG:dt

cc: Mr. J. P. O'Reilly, Region II  
Robert Lowenstein, Esquire