NRC FORM 195 U.S. NUCLEAR REGULATORY COMMISSION (2,776)				50-250/25/	
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Mr. Victor Stello		Miami, `Fla. Robert E. Uhrig		DATE RECEIVED 6/27/77	
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NRC FORM 195 (2-76)					





June 23, 1977 L-77-191



Director of Nuclear Reactor Regulation
Attention: Mr. Victor Stello, Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington DC 20555

Dear Mr. Stello:

Re:



File Cy.

Dear Mr. Sterro.

Turkey Point Units 3 & 4
Docket Nos. 50-250 and 50-251
Proposed Amendment to Facility
Operating Licenses DPR-31 and DPR-41

In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Appendix A of Facility Operating Licenses DPR-31 and DPR-41.

This proposal is being submitted to satisfy a commitment made in Reportable Occurrence report 250-77-5 of May 2, 1977, (attached). The proposed change is described below and shown on the accompanying Technical Specification page bearing the date of this letter in the lower right hand corner.

Figure 2.1-1

An analysis performed by our NSSS vendor has resulted in a corrected Figure 2.1-1 applicable at the licensed steady state reactor core thermal power output of 2200 Mwt.

The proposed amendment has been reviewed by the Turkey Point Plant Nuclear Safety Committee (PNSC) and the Florida Power & Light Company Nuclear Review Board (CNRB). They have concluded that it does not involve an unreviewed safety question.

Very truly yours,

Robert E. Uhrig Vice President

REU:MAS:tm Attachments

cc: Mr. Norman C. Moseley, Region II Robert Lowenstein, Esquire

771800049

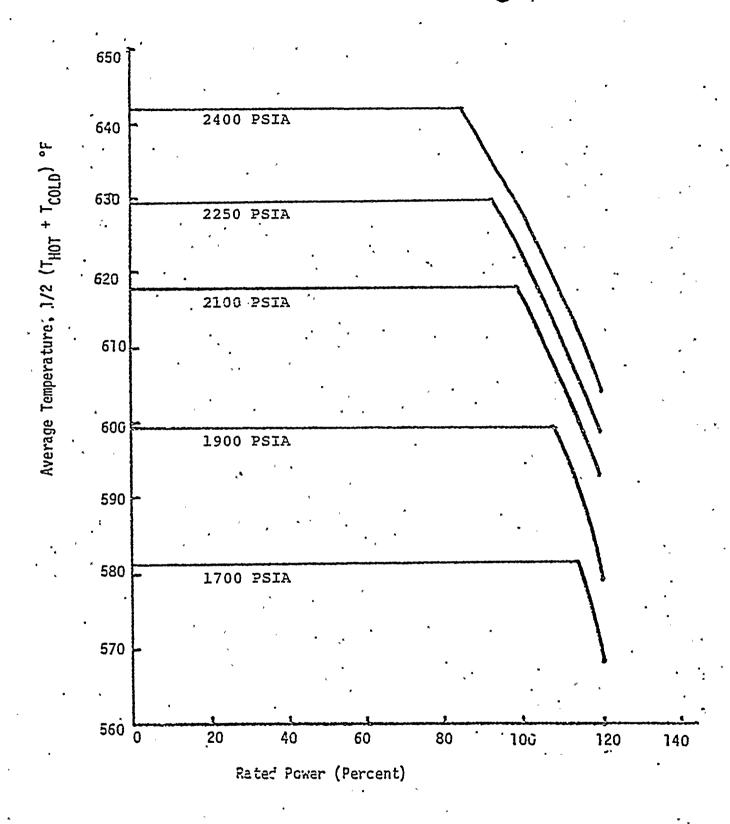


Fig. 1: 2.1-1. Reactor Core Thermal and Hydraulic Safety Limits, Three Loop Operation.



May 2, 1977

PRN-LI-77-131

Mr. Norman C. Moseley, Director, Region II Office of Inspection and Enforcement ...
U. S. Nuclear Regulatory Commission 230 Peachtree Street, N. W., Suite 1217 Atlanta, Georgia 30303

Dear Mr. Moseley:

REPORTABLE OCCURRENCE 250-77-5
TURKEY POINT UNIT 3
DATE OF OCCURRENCE: APRIL 18, 1977

NOMINAL POWER

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9.2 to provide prompt notification of the subject occurrence.

Very truly yours,

A. D. Schmidt
Vice President
Power Resources

MAS/cpc

Attachment

cc: Robert Lowenstein, Esquire
Director, Office of Inspection and Enforcement (40)
Director, Office of Management Information and
Program Control (3)

	LIGENSEE EVENT REPORT	
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01 7 8	CATEGORY TYPE SOURCE COCKET NUMBER - EVENT DATE DIT CONT T L 0 5 0 - 0 2 5 0 0 4 1 8 7 S 57 58 59 60 61 68 59	7 . 0 4 2 9 7 7 74 . 75 EO
	EVENT DESCRIPTION	
OIZ	Discussions with our NSSS vendor have shown that Technical Specific	
	"Reactor Core Thermal and Hydraulic Safety Limits", requires revis	
回	performed at 2200 Mwt has vielded a corrected Figure 2.1-1 application	
	and 4. No changes to setpoints, operating limits, or operating p	ractices are required
	_ = 5 S as a result of this correction. Neither unit has ever operated no	nconservatively as a
7 8	8 9 PRIVE CAUSE COMPONENT COMPONENT	
		ADAN N N
•	CAUSE DESCRIPTION.	
E	This occurrence was caused by a difference between the values of	
; Els;	Terror Turkey Point Units 3 and 4 by FPL and the NSSS vendor. The ve	ndor performs analysas
	E lossed on a nominal power of 2200 Myr. our licensed steady state r	80
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REPORTABLE OCCURRENCE 250-77-5 LICENSEE EVENT REPORT PAGE TWO

Event Description (continued)

result of the minor discrepancy in Figure 2.1-1. This was the first reportable occurrence at Turkey Point Unit 3 involving a nonconservative specification on reactor core thermal and hydraulic safety limits. A similar report (251-77-3) will be submitted for Unit 4.

(250-77-5)

Cause Description (continued)

power output. However, they recently advised us that the specific number being used for analytical work was 2192 Mwt. An analysis performed at 2200 Mwt showed that, in a single case, it would be necessary to make a minor administrative change to a Technical Specification (Figure 2.1-1). Figure 2.1-1 has been adjusted downward the necessary amount (approximately 1.5°F) to compensate for the correction. A proposed Technical Specification amendment will be submitted to the NRC to request a formal change correcting Figure 2.1-1.

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