

NRC FORM 195  
(2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER  
50-250(251)

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:  
V. STELLO

FROM:  
FLORIDA POWER & LIGHT CO.  
MIAMI, FLORIDA  
R. E. UHRIG

DATE OF DOCUMENT  
6-23-77

DATE RECEIVED  
7-1-77

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DESCRIPTION

Ltr. submitting in accordance with  
10 CFR 50.30 Request to amend Appendix A  
of Facility Operating Licenses DPR-31  
And 41 to satisfy a commitment made  
in RO 50-250/77-5 of 5/2/77.

ENCLOSURE

Figure 2.1-1 of tech specs

PLANT NAME:

*Corrected*  
*7/11/77* *988*

50-250/251

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DESCRIPTION

REQUEST TO AMEND APPENDIX A OF FACILITY  
OPERATING LICENSES DPR-31 AND DPR-41...  
W/ATTACHMENTS

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: TURKEY POINT # 3 & 4  
SAB

SAFETY

FOR ACTION/INFORMATION

ENVIRONMENTAL

ASSIGNED AD:

ASSIGNED AD: V. MOORE (LTR)

BRANCH CHIEF:

BRANCH CHIEF:

PROJECT MANAGER:

PROJECT MANAGER:

LICENSING ASSISTANT:

LICENSING ASSISTANT:

Leav (S)  
Ellis EC  
Pavish

B. HARLESS

INTERNAL DISTRIBUTION

REG FILES	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY &
NRG PDR	HEINEMAN	TEDESCO	ENVIRON ANALYSIS
T & E (2)	SCHROEDER	BENAROYA	DENTON & MULLER
OELD		LAINAS	CRITCHFIELD
GOSSICK & STAFF	ENGINEERING	IPPOLITO	
HANAUER	KNIGHT	F. ROSA	ENVIRO TECH.
MTPC	BOSNAK		ERNST
CASE	SIHWELL	OPERATING REACTORS	BALLARD
BOYD	PAWLICKI	STELLO	YOUNGBLOOD
		EISENHUT	
PROJECT MANAGEMENT	REACTOR SAFETY	SHAO	
SKOVHOLT	ROSS	BAER	SITE TECH.
P. COLLINS	NOVAK	BUTLER	GAMMILL (2)
HOUSTON	ROSZTOCZY	GRIMES	
MELTZ	CHECK		SITE ANALYSIS
HELTEMES			VOLLMER
SK	AT&I		BUNCH
	SALTZMAN		J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: Main Pla

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REG IV (J. HANCHETT)

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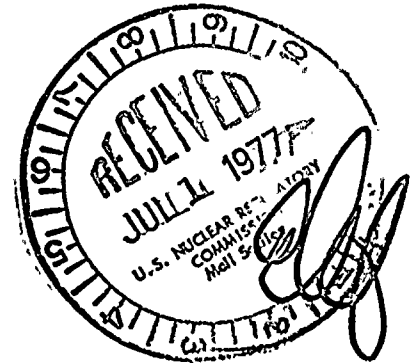
Apple  
771860096

RegulatoryFile CysJune 23, 1977  
L-77-191

Director of Nuclear Reactor Regulation  
 Attention: Mr. Victor Stello, Director  
 Division of Operating Reactors  
 U. S. Nuclear Regulatory Commission  
 Washington DC 20555

Dear Mr. Stello:

Re: Turkey Point Units 3 & 4  
 Docket Nos. 50-250 and 50-251  
 Proposed Amendment to Facility  
Operating Licenses DPR-31 and DPR-41



In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Appendix A of Facility Operating Licenses DPR-31 and DPR-41.

This proposal is being submitted to satisfy a commitment made in Reportable Occurrence report 250-77-5 of May 2, 1977, (attached). The proposed change is described below and shown on the accompanying Technical Specification page bearing the date of this letter in the lower right hand corner.

Figure 2.1-1

An analysis performed by our NSSS vendor has resulted in a corrected Figure 2.1-1 applicable at the licensed steady state reactor core thermal power output of 2200 Mwt.

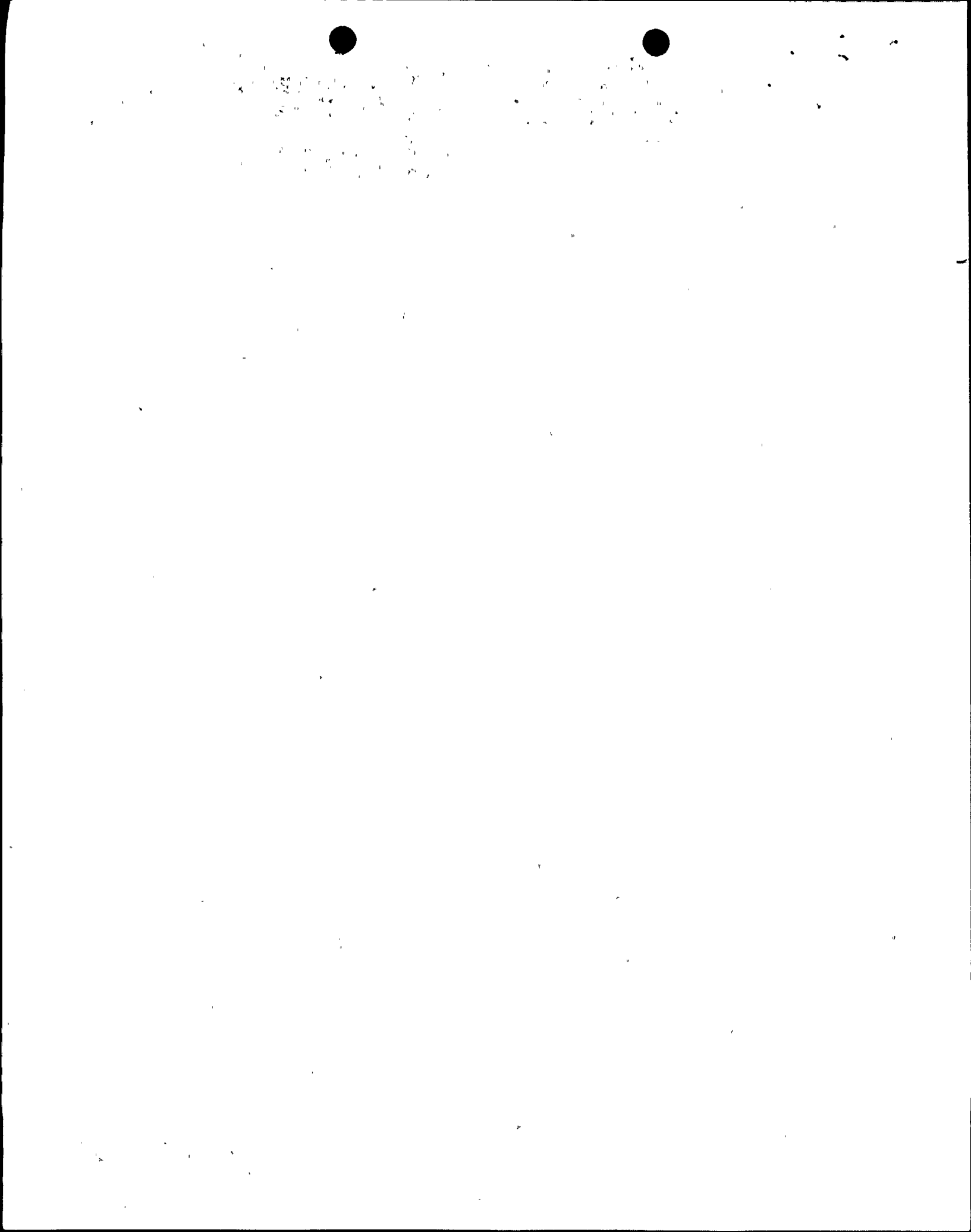
The proposed amendment has been reviewed by the Turkey Point Plant Nuclear Safety Committee (PNSC) and the Florida Power & Light Company Nuclear Review Board (CNRB). They have concluded that it does not involve an unreviewed safety question.

Very truly yours,

Robert E. Uhrig  
 Vice President

REU:MAS:tm  
 Attachments

cc: Mr. Norman C. Moseley, Region II  
 Robert Lowenstein, Esquire



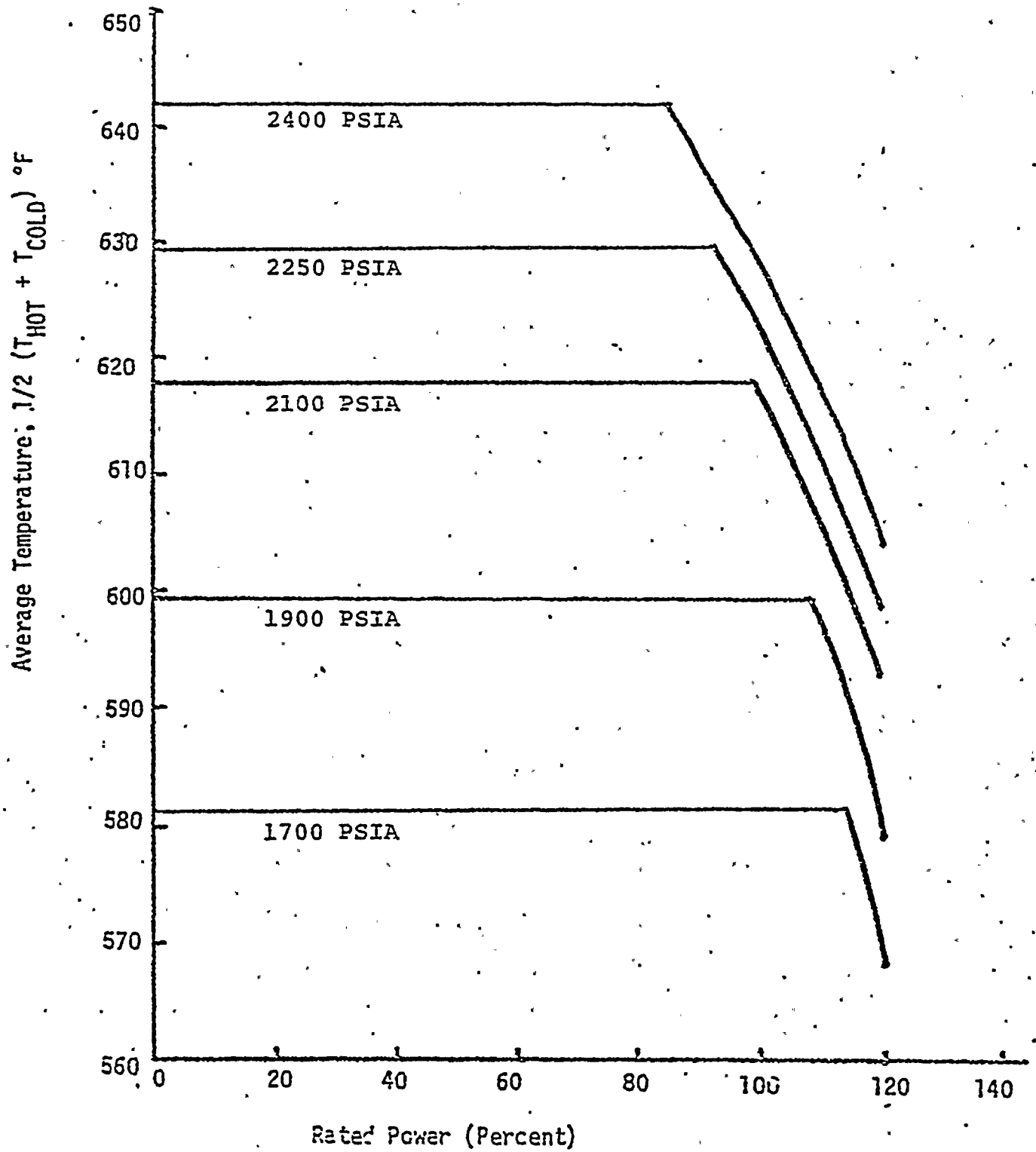
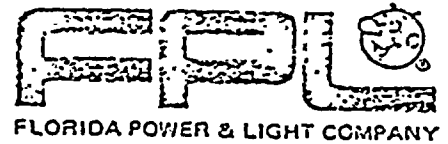


Fig. 2.1-1. Reactor Core Thermal and Hydraulic Safety Limits, Three loop Operation

6/23/77



May 2, 1977

PRN-LI-77-131

Mr. Norman C. Moseley, Director, Region II  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
230 Peachtree Street, N. W., Suite 1217  
Atlanta, Georgia 30303

Dear Mr. Moseley:

REPORTABLE OCCURRENCE 250-77-5  
TURKEY POINT UNIT 3  
DATE OF OCCURRENCE: APRIL 18, 1977

NOMINAL POWER

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9.2 to provide prompt notification of the subject occurrence.

Very truly yours,

*J.R. Bensen*

A. D. Schmidt  
Vice President  
Power Resources

MAS/cpc

Attachment

cc: Robert Lowenstein, Esquire  
Director, Office of Inspection and Enforcement (40)  
Director, Office of Management Information and  
Program Control (3)

CONTROL BLOCK: 1 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME

LICENSE NUMBER

LICENSE TYPE

EVENT TYPE

01 | F | L | T | P | S | 3 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 4 | 1 | 1 | 1 | 1 | 0 | 1 |

01 | CGNT | 57 | 58 | T | L | 0 | 5 | 0 | - | 0 | 2 | 5 | 0 | 0 | 4 | 1 | 8 | 7 | 7 | 0 | 4 | 2 | 9 | 7 | 7 |

EVENT DESCRIPTION

012 | Discussions with our NSSS vendor have shown that Technical Specification Figure 2.1-1.
013 | "Reactor Core Thermal and Hydraulic Safety Limits", requires revision. An analysis
014 | performed at 2200 Mwt has yielded a corrected Figure 2.1-1 applicable to both Units 3
and 4. No changes to setpoints, operating limits, or operating practices are required
as a result of this correction. Neither unit has ever operated nonconservatively as a

017 | SYSTEM CODE: Z | Z | CAUSE CODE: F | COMPONENT CODE: Z | Z | Z | Z | Z | Z | PRIMARY COMPONENT SUPPLIER: Z | COMPONENT MANUFACTURER: Z | 9 | 9 | 9 | VIOLATION: N

CAUSE DESCRIPTION

018 | This occurrence was caused by a difference between the values of "nominal power" applied
to Turkey Point Units 3 and 4 by FPL and the NSSS vendor. The vendor performs analyses
based on a nominal power of 2200 Mwt, our licensed steady state reactor core thermal

019 | FACILITY STATUS: E | % POWER: 1 | 0 | 0 | OTHER STATUS: NA | METHOD OF DISCOVERY: d | DISCOVERY DESCRIPTION: NSSS Vendor Discussions

020 | FORM OF ACTIVITY RELEASED: Z | CONTENT OF RELEASE: Z | AMOUNT OF ACTIVITY: NA | LOCATION OF RELEASE: NA

021 | PERSONNEL EXPOSURES: NUMBER: 0 | 0 | 0 | TYPE: Z | DESCRIPTION: NA

022 | PERSONNEL INJURIES: NUMBER: 0 | 0 | 0 | DESCRIPTION: NA

023 | PROBABLE CONSEQUENCES: NA

024 | LOSS OR DAMAGE TO FACILITY: TYPE: Z | DESCRIPTION: NA

025 | FUELILITY: NA

ADDITIONAL FACTORS

026 | See Page Two for continuation of Event Description and Cause Description.

027 |

Event Description (continued)

result of the minor discrepancy in Figure 2.1-1. This was the first reportable occurrence at Turkey Point Unit 3 involving a nonconservative specification on reactor core thermal and hydraulic safety limits. A similar report (251-77-3) will be submitted for Unit 4.

(250-77-5)

Cause Description (continued)

power output. However, they recently advised us that the specific number being used for analytical work was 2192 Mwt. An analysis performed at 2200 Mwt showed that, in a single case, it would be necessary to make a minor administrative change to a Technical Specification (Figure 2.1-1). Figure 2.1-1 has been adjusted downward the necessary amount (approximately 1.5°F) to compensate for the correction. A proposed Technical Specification amendment will be submitted to the NRC to request a formal change correcting Figure 2.1-1.



