

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:  
  
Mr. Victor Stello, Jr.

FROM:  
Florida Power & Light Company  
Miami, Florida  
Mr. Robert E. Uhrig

DATE OF DOCUMENT  
8/25/76

DATE RECEIVED  
8/27/76

LETTER  
 ORIGINAL  
 COPY

NOTORIZED  
 UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED  
One signed

DESCRIPTION

Ltr. w/attached..re their 8/19/76 ltr.  
...furnishing Westinghouse Safety Analysis  
page correction...concerning a new Fq  
limit for Units 3 & 4.

PLANT NAME: (2-P)  
  
Turkey Point 3 & 4

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

SAFETY FOR ACTION/INFORMATION ENVIRO 8/27/76 RJL

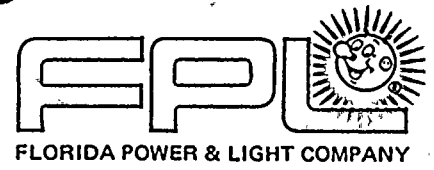
|                  |          |                  |
|------------------|----------|------------------|
| ASSIGNED AD:     |          | ASSIGNED AD:     |
| BRANCH CHIEF:    | Lear (6) | BRANCH CHIEF:    |
| PROJECT MANAGER: | Elliott  | PROJECT MANAGER: |
| LIC. ASST.:      | Parrish  | LIC. ASST.:      |
|                  | DeYoung  |                  |

INTERNAL DISTRIBUTION

|   |                |                    |                 |
|---|----------------|--------------------|-----------------|
| <input checked="" type="checkbox"/> REG FILE        | SYSTEMS SAFETY | PLANT SYSTEMS      | SITE SAFETY &   |
| <input checked="" type="checkbox"/> NRC PDR         | HEINEMAN       | TEDESCO            | ENVIRO ANALYSIS |
| <input checked="" type="checkbox"/> I & E (2)       | SCHROEDER      | BENAROYA           | DENTON & MULLER |
| <input checked="" type="checkbox"/> OELD            |                | LAINAS             |                 |
| <input checked="" type="checkbox"/> GOSSICK & STAFF | ENGINEERING    | IPPOLITO           | ENVIRO TECH.    |
| <input checked="" type="checkbox"/> MIPC            | MACCARRY       | KIRKWOOD           | ERNST           |
| <input checked="" type="checkbox"/> CASE            | KNIGHT         |                    | BALLARD         |
| <input checked="" type="checkbox"/> HANAUER         | SINWEIL        | OPERATING REACTORS | SPANGLER        |
| <input checked="" type="checkbox"/> HARLESS         | PAWLICKI       | STELLO             |                 |
|   |                |                    | SITE TECH.      |
| PROJECT MANAGEMENT                                  | REACTOR SAFETY | OPERATING TECH.    | GAMMILL         |
| <input checked="" type="checkbox"/> BOYD            | ROSS           | EISENHUT           | STAPP           |
| <input checked="" type="checkbox"/> P. COLLINS      | NOVAK (3)      | SHAO               | HULMAN          |
| <input checked="" type="checkbox"/> HOUSTON         | ROSZTOCZY      | BAER               |                 |
| <input checked="" type="checkbox"/> PETERSON        | CHECK          | BUTLER             | SITE ANALYSIS   |
| <input checked="" type="checkbox"/> MELTZ           |                | GRIMES             | VOLLMER         |
| <input checked="" type="checkbox"/> HELTEMES        | AT & I         |                    | BUNCH           |
| <input checked="" type="checkbox"/> SKOVHOLT        | SALTZMAN       |                    | J. COLLINS      |
|   | RUTBERG        |                    | KEEGER          |

EXTERNAL DISTRIBUTION

|   |             |                    |                            |
|---|-------------|--------------------|----------------------------|
| <input checked="" type="checkbox"/> I.PDR: Miami, Florida.            | NAT LAB:    | BROOKHAVEN NAT LAB | CONTROL NUMBER<br><br>8716 |
| <input checked="" type="checkbox"/> TIC:                              | REG. VIE    | ULRIKSON (ORNL)    |                            |
| <input checked="" type="checkbox"/> NSIC:                             | IA PDR      |                    |                            |
| <input checked="" type="checkbox"/> ASLB:                             | CONSULTANTS |                    |                            |
| <input checked="" type="checkbox"/> ACRS/6 CYS HOLDING/SENT: PARRISH. |             |                    |                            |
|   |             |                    |                            |



August 25, 1976  
L-76-307

Director of Nuclear Reactor Regulation  
Attn: Victor Stello, Jr., Director  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555



Dear Mr. Stello:

Re: Turkey Point Units 3 and 4  
Docket Nos. 50-250 and 50-251  
Westinghouse Safety Analysis Page Correction

The Florida Power & Light Company wishes to submit a revised page 1 to our letter of August 19, 1976, L-76-300, which discussed a new F<sub>0</sub> limit for Turkey Point Units 3 and 4. A copy of the revised page is attached.

Very truly yours,

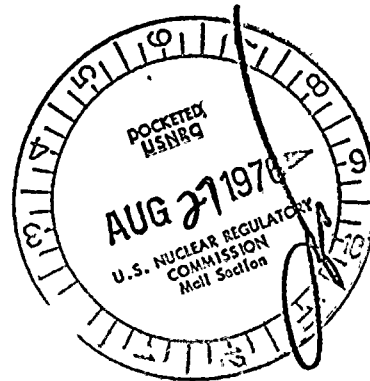
*J.A. De Mastroy*

Robert E. Uhrig  
Vice President

REGULATORY DOCKET FILE COPY

REU/JAD/hlc  
Attachment

cc: Norman C. Moseley, Region II  
Jack R. Newman, Esq.



8716



FLORIDA POWER &amp; LIGHT COMPANY

August 19, 1976

L-76-300

Office of Nuclear Reactor Regulation  
 Attn: Victor Stello, Jr., Director  
 Division of Operating Reactors  
 U. S. Nuclear Regulatory Commission  
 Washington, D. C. 20555

Dear Mr. Stello:

Re: Turkey Point Units 3 and 4  
 Docket Nos. 50-250 and 50-251  
Westinghouse Safety Analyses

The Westinghouse Electric Corporation has informed Florida Power & Light Company that recent developments have affected the results of certain safety analyses for Westinghouse plants. The following information regarding this problem is being submitted to you in response to an August 13, 1976 telephone request from your staff.

One development involves the temperature of the fluid in the upper head. Past ECCS analyses assumed that the temperature in the upper head was equal to the vessel inlet temperature ( $T_{cold}$ ). The conservative judgment is to assume that the temperature will be equal to the vessel outlet temperature ( $T_{hot}$ ). The consequence is a reduction in maximum allowable Heat Flux Hot Channel Factor ( $F_Q$ ) to prevent operation at unacceptable local power levels. The resulting  $F_Q$  limit for Turkey Point is conservatively estimated to be 2.11. The new  $F_Q$  limit was derived as shown below:

$$[2.32 - A - B + C] D = 2.11$$

2.32 = previous limit on  $F_Q$

A = .31 = estimated reduction in  $F_Q$  due to increase in upper head temperature from  $T_{cold}$  to  $T_{hot}$ .

B = .04 = estimated reduction in  $F_Q$  due to plugged steam generator tubes.

C = .05 = estimated increase in  $F_Q$  due to the fact that the ECCS analysis peak clad temperature is 50° below the Final Acceptance Criteria.

D =  $\frac{2300}{2200}$  = factor for increasing  $F_Q$  due to operation 100 Mwt below the power level used in the ECCS analysis.

Note: For a 925 ft<sup>3</sup> accumulator volume, A = .16 and  $F_Q = 2.27$ .

