NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

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TO: MR D L ZIEMANN

FROM: COMMONWEALTH EDISON CHICAGO, ILL
G A ABRELL

DATE OF DOCUMENT: 2-26-76

DATE RECEIVED: 3-1-76

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INPUT FORM

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DESCRIPTION

LTR...PROVIDING TABLE #2 REVISED IN FEB 76 PERTAINING TO THE LATEST CORE WIDE METAL TO WATER REACTION PERCENTAGE.....

THIS VALUE ALSO APPLIES TO DRESDEN 1 & 2.....

ENCLOSURE

PLANT NAME: QUAD CITIES 1 & 2

FOR ACTION/INFORMATION — ENVIRO 3-4-76 RB

	SAFETY				
	ASSIGNED AD :			ASSIGNED AD :	
X	BRANCH CHIEF : (6)	ZIEMANN		BRANCH CHIEF :	
	PROJECT MANAGER:			PROJECT MANAGER :	
X	LIC. ASST. :	Diggs		LIC. ASST. :	

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	HANAUER		SIHWEIL		STELLO		STEPP
	HARLESS		PAWLICKI				HULMAN
					OPERATING TECH		
	PROJECT MANAGEMENT		REACTOR SAFETY	X	EISENHUT		SITE ANALYSIS
	BOYD		ROSS	X	SHAO		VOLLMER
	P. COLLINS		NOVAK	X	BAER		BUNCH
	HOUSTON		ROSZTOCZY	X	SCHWENCER		J. COLLINS
	PETERSON		CHECK	X	GRIMES		KREGER
	MELTZ						
	HELTEMES		AT & I		SITE SAFETY & ENVIRO		
	SKOVHOLT		SALTZMAN		ANALYSIS		
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CONTROL NUMBER: 1959

NRC FORM 195 (2-76)

Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

February 26, 1976

Mr. D. L. Ziemann, Chief
Operating Reactors - Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Quad-Cities Station Special Report No. 15, Supplement C
NRC Docket No. 50-254 and 50-265

Dear Mr. Ziemann:

Attached is Table 2 revised in February 1976. The revised table indicates the latest core wide metal to water reaction percentage of .18. This value also applies to Dresden Station Units 2 and 3 in that various submittals for these units refer to Quad-Cities Special Report, Supplement C.

One (1) signed original and 39 copies are enclosed for your use.

Very truly yours,

G. A. Abrell
Nuclear Licensing Administrator
Boiling Water Reactors

Enclosure

1959

TABLE 2

QUAD CITIES APPENDIX K RESULTS

	PCT (°F)	PEAK LOCAL OXIDATION %	CORE-WIDE METAL-WATER REACTION %	PCT(1)	OTHER SINGLE FAILURES CONSIDERED(5)			
o Break size o Location o Single failure					LPCI INJ VLV	HPCI	DIESEL	1 ADS
o 4.2 ft^2 (DBA) o Recirc suction o LPCI Inj valve	2200 (2)	8	0.8	-	-	B,C	B	C
o 3.54 ft^2 (0.85 DBA) o Recirc suction o LPCI Inj valve	2090 (2)	2	-	-	-	B,C	B	C
o 2.5 ft^2 (0.6 DBA) o Recirc suction o LPCI Inj valve	1980 (2)	1	-	-	-	B,C	B	C
o 1.0 ft^2 o Recirc suction (Large Break Methods)	1650(2)	<1	-	-	-	B,C	B	C
o LPCI Inj valve (Small Break Methods)	1500(3)	<1	-	-	-	B,C	B	C
o 0.07 ft^2 o Recirc suction o HPCI	1730(2) 1800(3)	<1	-	-	C	-	C	C
o 0.03 ft^2 o Recirc suction o HPCI	1540(3)	<1	-	-	C	-	C	C
o 2.28 ft^2 o Main steam line o LPCI Inj valve	550(4)	<1	-	1950	-	B,C	B	C
o 0.50 ft^2 o Feedwater Line (Disables HPCI) o LPCI Inj valve	550(4)	<1	-	~1250	-	-	B	C
o 0.27 ft^2 o Core spray line (Disables 1 CS) o HPCI	1210(3)	<1	-	1400	A	-	B	C

NOTES

(1) PCT for recirc line break of same size, °F
(2) PCT from CHASTE
(3) PCT from non-DBA REFLOOD
(4) No uncovery
(5) Code for other single failures considered:
A. Specific App. K calculation, PCT less than that indicated
B. Relative ECCS pumping capacity much more than that for single failure considered, no need for specific calculation.
C. Experience with similar analyses indicates that this failure is less severe than those for which a specific calculation was conducted.