

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO: MR D L ZIEMANN

FROM: COMMONWEALTH EDISON
CHICAGO, ILL
G A ABRELL

DATE OF DOCUMENT
2-26-76

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3-1-76

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DESCRIPTION

LTR...PROVIDING TABLE #2 REVISED IN FEB 76
PERTAINING TO THE LATEST CORE WIDE
METAL TO WATER REACTION PERCENTAGE.....

THIS VALUE ALSO APPLIES TO DRESDEN 1 & 2.....

PLANT NAME: QUAD CITIES 1 & 2

ENCLOSURE

SAFETY		FOR ACTION/INFORMATION		ENVIRO	
ASSIGNED AD :		ASSIGNED AD :		3-4-76	RB
<input checked="" type="checkbox"/> BRANCH CHIEF : (6)	ZIEMANN	BRANCH CHIEF :			
PROJECT MANAGER:		PROJECT MANAGER :			
<input checked="" type="checkbox"/> LIC. ASST. :	Diags	LIC. ASST. :			

INTERNAL DISTRIBUTION			
<input checked="" type="checkbox"/> REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	ENVIRO TECH
<input checked="" type="checkbox"/> NRC PDR	HEINEMAN	TEDESCO	ERNST
<input checked="" type="checkbox"/> I & E (2)	SCHROEDER	BENAROYA	BALLARD
OELD		LAINAS	SPANGLER
GOSSICK & STAFF	ENGINEERING	IPPOLITO	
MIPC	MACCARY		SITE TECH
CASE	KNIGHT	OPERATING REACTORS	GAMMILL
HANAUER	SIHWEL	STELLO	STEPP
HARLESS	PAWLICKI		HULMAN
		OPERATING TECH	
PROJECT MANAGEMENT	REACTOR SAFETY	<input checked="" type="checkbox"/> EISENHUT	SITE ANALYSIS
BOYD	ROSS	<input checked="" type="checkbox"/> SHAO	VOLLMER
P. COLLINS	NOVAK	<input checked="" type="checkbox"/> BAER	BUNCH
HOUSTON	ROSZTOCZY	<input checked="" type="checkbox"/> SCHWENCER	J. COLLINS
PETERSON	CHECK	<input checked="" type="checkbox"/> GRIMES	KREGER
MELTZ			
HELTEMES	AT & I	SITE SAFETY & ENVIRO	
SKOVHOLT	SALTZMAN	ANALYSIS	
	RUTBERG	DENTON & MULLER	

EXTERNAL DISTRIBUTION		
<input checked="" type="checkbox"/> LPDR: Moline, Ill..	NATL LAB	BROOKHAVEN NATL LAB
<input checked="" type="checkbox"/> TIC	REG. V-IE	ULRIKSON(ORNL)
<input checked="" type="checkbox"/> NSIC	LA PDR	
ASLB	CONSULTANTS	
ACRS 16 HOLDING/SENT		

CONTROL NUMBER

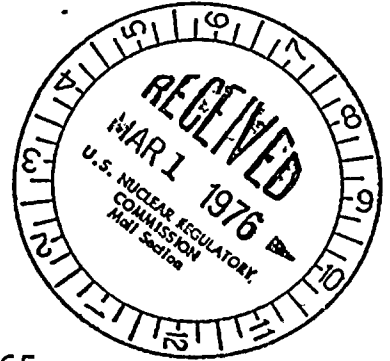
1959



Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

February 26, 1976

Mr. D. L. Ziemann, Chief
Operating Reactors - Branch 2
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Subject: Quad-Cities Station Special
Report No. 15, Supplement C
NRC Docket No. 50-254 and 50-265

Dear Mr. Ziemann:

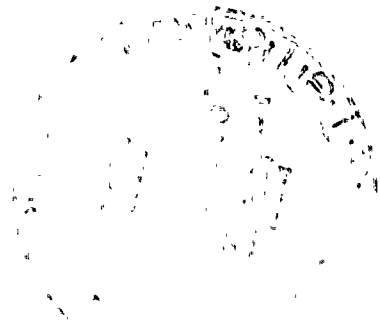
Attached is Table 2 revised in February 1976. The revised table indicates the latest core wide metal to water reaction percentage of .18. This value also applies to Dresden Station Units 2 and 3 in that various submittals for these units refer to Quad-Cities Special Report, Supplement C.

One (1) signed original and 39 copies are enclosed for your use.

Very truly yours,

G. A. Abrell
Nuclear Licensing Administrator
Boiling Water Reactors

Enclosure



1959



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TABLE 2
QUAD CITIES APPENDIX K RESULTS

Break size Location Single failure	PCT (°F)	PEAK LOCAL OXIDATION %	CORE-WIDE METAL-WATER REACTION %	PCT (1)	OTHER SINGLE FAILURES CONSIDERED (5)			
					LPCI INJ VLV	HPCI	DIESEL	1 ADS
4.2 ft ² (DBA) Recirc suction LPCI Inj valve	2200 (2)	8	0.8	-	-	B,C	B	C
3.54 ft ² (0.85 DBA) Recirc suction LPCI Inj valve	2090 (2)	2	-	-	-	B,C	B	C
2.5 ft ² (0.6 DBA) Recirc suction LPCI Inj valve	1980 (2)	1	-	-	-	B,C	B	C
1.0 ft ² Recirc suction Large Break Methods	1650 (2)	<1	-	-	-	B,C	B	C
LPCI Inj valve Small Break Methods	1500 (3)	<1	-	-	-	B,C	B	C
0.07 ft ² Recirc suction HPCI	1730 (2) 1890 (3)	<1	-	-	C	-	C	C
0.03 ft ² Recirc suction HPCI	1540 (3)	<1	-	-	C	-	C	C
2.28 ft ² Main steam line LPCI Inj valve	550 (4)	<1	-	1950	-	B,C	B	C
0.50 ft ² Feedwater Line (Disables HPCI) LPCI Inj valve	550 (4)	<1	-	~1250	-	-	B	C
0.27 ft ² Core spray line (Disables 1 CS) HPCI	1210 (3)	<1	-	1400	A	-	B	C

NOTES

- (1) PCT for recirc line break of same size, °F
- (2) PCT from CHASTE
- (3) PCT from non-DBA REFLOOD

(4) No uncover

(5) Code for other single failures considered:

- A. Specific App. K calculation, PCT less than that indicated
- B. Relative EGCS pumping capacity much more than that for single failure considered, no need for specific calculation.
- C. Experience with similar analyses indicates that this failure is less severe than those for which a specific calculation was conducted.