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(TEMPORARY FORM)

CONTROL NO: 10176

FILE:

FROM: Florida Power & Light Co Miami, Fla. 33101 Mr. R.E. Uhrig	DATE OF DOC 9-27-74	DATE REC'D 10-2-74	LTR X	TWX	RPT	OTHER
TO: E.G. Case	ORIG 3 signed	CC	OTHER	SENT AEC PDR XXX SENT LOCAL PDR XXX		
CLASS	UNCLASS XXX	PROP INFO	INPUT XXX	NO CYS REC'D 40	DOCKET NO: 50-250/251	

DESCRIPTION:

Ltr notarized 9-27-74...trans the following.. .....

ENCLOSURES:

Proposed changes to the tech specs...

ACKNOWLEDGED

(40 cys encl rec'd)

PLANT NAME: Turkey Point 3 & 4

DO NOT REMOVE

FOR ACTION/INFORMATION 10-8-74 JB

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CASE	KNIGHT	BALLARD	GOULBOURNE (L)	
GIAMBUSSO	PAWLICKI	SPANGLER	KREUTZER (E)	PLANS
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MOORE (L) (BWR)	STELLO	ENVIRO	MAIGRET (L)	CHAPMAN
DEYOUNG (L) (PWR)	HOUSTON	MULLER	REED (E)	✓ DUBE w/input
SKOVHOLT (L)	NOVAK	DICKER	SERVICE (L)	✓ E. COUPE
✓ GOLLER (L)	ROSS	KNIGHTON	SHEPPARD (L)	✓ Schemel
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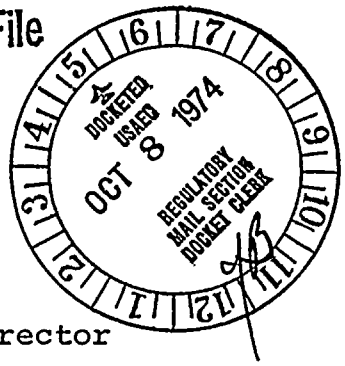
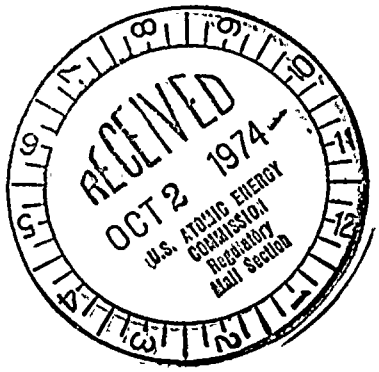
P.O. BOX 3100 MIAMI, FLORIDA 33101

Regulatory Docket File



FPL
FLORIDA POWER & LIGHT COMPANY

September 27, 1974



Mr. Edson G. Case, Acting Director
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. Case:

Re: TURKEY POINT PLANT UNITS 3 AND 4
DOCKET NOS. 50-250 AND 50-251
PROPOSED AMENDMENT TO THE
FACILITY OPERATING LICENSES
DPR-31 AND DPR-41

In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three signed originals and forty (40) conformed copies of a request to amend the Facility Operating Licenses DPR-31 and DPR-41.

This submittal is in response to Mr. Karl Goller's letter of July 18, 1974, in which we were requested to submit a proposed change to our Technical Specifications establishing requirements for a program of steam generator tube inspection. On August 28, 1974, we advised Mr. Goller that we were working with the reactor vendor in this regard and we would submit an inspection program by October 1, 1974.

The program which we have established essentially conforms with the level of inspection required by Regulatory Guide 1.83, Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes. However, the program does not address the subject of baseline inspection. We believe that the extensive inspections performed on the Unit No. 4 steam generators constitutes a sufficient basis for baseline inspections. Based on the Unit No. 4 results, the Unit No. 3 steam generators will also be extensively inspected during the upcoming refueling outage.

The changes we are requesting are shown on the attached revised pages of the Technical Specifications (Appendix A of the Facility Operating Licenses). These changes consist of a revision to Table 4.2-1 and to Section B4.2 of the Bases.

We have reviewed this program and have concluded that it meets or exceeds the requirements of Regulatory Guide 1.83. We have also

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concluded that it does not involve a significant hazards consideration and there is reasonable assurance that the health and safety of the public will not be endangered.

Very truly yours,

Robert E. Uhrig
Vice President

REU/DWR/cpc

Attachment

cc: Jack R. Newman, Esquire

STATE OF FLORIDA)
)
COUNTY OF DADE)

ROBERT E. UHRIG, being first duly sworn, deposes and says:

That he is a Vice President of Florida Power & Light Company, the Licensee herein;

That he has executed the foregoing instrument; that the statements made in this said instrument are true and correct to the best of his knowledge, information and belief; and that he is authorized to execute the instrument of said Licensee.

Robert E. Uhrig

Subscribed and sworn to before me
this 27th day of September, 1974.

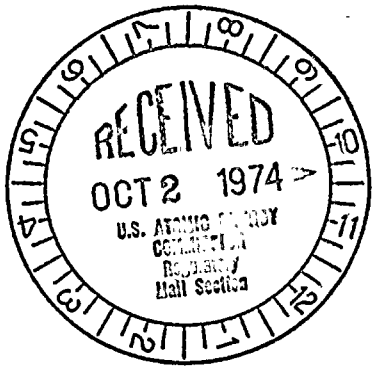
Harriet E. Wilson
Notary Public in and for the
County of Dade, State of Florida

Notary Public, State of Florida at Large
My Commission Expires Oct. 30, 1974
Bonded by American Fire & Casualty Co.

BOX 013100, MIAMI, FLORIDA 33101

FLORIDA POWER & LIGHT COMPANY

September 27, 1974



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County of Dade, State of Florida

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My Commission Expires Oct. 30, 1974
Bonded by American Fire & Casualty Co.

by a factor of 2.1. Thus, this capsule provides information for approximately a four-year exposure to the vessel.

Capsule No. 2 is scheduled to be removed at the fourth region replacement. This capsule leads the vessel maximum exposure by a factor of 0.8 and thus will provide data for a four-year exposure to the vessel. This sample also contains weld metal which is not present in Capsule No. 1.

Capsule No. 3 leads the vessel maximum exposure by a factor of 2.2 and is scheduled to be removed after twenty years. Thus, sample No. 3 will provide data for an exposure to the vessel of approximately forty years.

Capsules No. 4 and 5 lead the maximum vessel exposure by factors of 0.7 and 0.5, respectively. Thus, Capsule No. 4, which is scheduled to be removed after thirty years, provides data for a vessel exposure of twenty-one years and Capsule No. 5, which is scheduled to be removed at forty years, provides data for a vessel exposure of twenty years.

In addition to the capsules discussed above, there are three spares.

Item 7.3 - Steam Generator Tubes

1) Sample Selection and Testing - Each steam generator shall be inspected during shutdown as follows:

   a) During each inservice inspection (at frequencies specified in Item 7.3.2), a representative sample of at least 3 percent of the total number of steam generator tubes shall be eddy current inspected. Inspections subsequent to the first two inservice inspections of the steam generator tubes shall routinely concentrate on the hot-leg side of the steam generator.

      The tubes selected for each inservice inspection shall include:

1. All tubes (except plugged tubes) in which wall penetrations of >20 percent were revealed during previous inspections.

2. Tubes in those areas (including the cold-leg side) where design and experience have indicated potential problems.

b) As a minimum, an additional 3 percent of the steam generator tubes shall be eddy current inspected if more than 10 percent (all tubes previously identified as having wall penetrations >20 percent and exhibiting further wall penetrations of >5 percent shall be included in this 10 percent) of the tubes inspected per 1)a above, have wall penetrations of >20 percent or if one or more of the inspected tubes require plugging as defined in 3)a. At least 75 percent of these additional tubes selected for eddy current inspection shall be from tubes in those areas of the tube sheet array where tubes with defects were found.

c) As a minimum, an additional 3 percent of the steam generator tubes shall be eddy current inspected if more than 10 percent of the tubes inspected per 1)b above, have wall penetrations of >20 percent or if one or more of the inspected tubes require plugging as defined in 3)a. These additional tubes selected for eddy current inspection shall be from tubes in those areas of the tube sheet array where tubes with defects were found.

2) Inspection Frequencies - The above inservice inspections of steam tubes shall be performed at the following frequencies:

a) At intervals of not less than 12 or more than 20 calendar months, except the first scheduled inservice inspection shall be performed at the first extended outage after 6 months of operation. If two consecutive inservice inspections of steam generator tubes show no additional tubes with wall penetrations of >20 percent and no significant (>5 percent) further penetration of tubes with previous defect indications, the inspection frequency for hot-leg tubes may be extended to at least once per 40 months. If two consecutive

inservice inspections of the cold-leg tubes of the steam generator show no additional tubes with wall penetrations >20 percent and no significant (>5 percent) further penetration of tubes with previous defect indications, the inspection frequency for cold-leg tubes may be extended to at least once per 10 years.

b) During the shutdown subsequent to any of the following conditions:

1. Primary-to-secondary leakage in excess of 1 gpm.

2. A seismic occurrence greater than the Operating Basis Earthquake.

3. A loss-of-coolant accident requiring actuation of the engineered safeguards.

4. A steam line or feed water line break requiring a reactor shutdown.

3) Acceptance Criteria

a) Any tubes with an eddy current indication of 50% or greater wall penetration shall be plugged before the steam generator is returned to service.

b) If in the inspections performed under Item 7.3.1, less than 10% of the total tubes inspected have detectable wall penetration (>20%) and no tubes require plugging per Acceptance Criterion 3)a, plant operation shall be resumed and the inspection results shall be reported in the semi-annual operating reports covering the periods of operation in which these inspections were completed.

c) If in the inspections performed under Item 7.3.1, less than 10% of the total tubes inspected have detectable wall penetration (>20%) and no more than 3 tubes require plugging per Acceptance

Criterion 3)a, plant operation shall be resumed after corrective action given in Acceptance Criterion 3)b has been taken. The results of these inspections and corrective actions shall be reported in the semi-annual operating reports covering the periods of operation in which these inspections were completed.

d) If in the inspections performed under Item 7.3.1, more than 10% of the total tubes inspected have detectable wall penetrations (>20%), or more than 3 of the tubes inspected require plugging, the situation will be assessed by the plant operator and appropriate action will be taken prior to plant operation and the event shall be reported to the Commission as an abnormal occurrence.

TABLE 4.2-1 (cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Method	Extent of Examination (Percent in 10 Year Interval)	Extent of Examination (Percent in 5 Year Interval)	Remarks
6.5	G-2	Pressure-retaining bolt	Visual and Volumetric	100%	33%	Exception is taken for valves which are not accessible.
6.6	K-1	Integrally-welded supports		Not Applicable	Not Applicable	
6.7	K-2	Supports and Hangers	Visual	100%	33%	Exception is taken for supports and hangers which are not accessible.
7.1		Reactor coolant pump Flywheel	V & UT	100% for (2)	In place at bore, Keyway, and tapped holes for (1).	The flywheels shall be visually examined at the first refueling and at the end of each 10 year interval. The outside surface shall be examined by ultrasonic methods on the following schedule: (1) During second refueling - all 3, (2) At the seventh, eighth and ninth refueling - one different flywheel each refueling (3) Repeat (2) within 10 years.
7.2		Irradiation Specimen Schedule	Tensile and Charpy V Notch (Wedge Open Loading)	See Remarks	See Remarks	Capsule 1 shall be removed and examined at the first region replacement. Capsule 2 shall be removed and examined at the fourth region replacement. Capsule 3 shall be removed and examined after twenty years of operation. Capsule 4 shall be removed and examined after thirty years of operation. Capsule 5 shall be removed and examined after forty years of operation
7.3		Steam Generator Tubes	Eddy Current	See Bases	See Bases	See Bases