

A 10/06/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-250

REC: STELLO V
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DOCDATE: 10/02/78
DATE RCVD: 10/06/78

DOCTYPE: LETTER NOTARIZED: YES

COPIES RECEIVED
LTR 3 ENCL 40

SUBJECT: FURNISHING SUPPLEMENTAL INFO TO APPLICANT'S 09/26/78 TECH SPEC PROPOSED AMEND TO LIC NO. DPR-31 TO ALLOW UNIT 3 TO OPERATE UNTIL THE NEXT REFUELING PRIOR TO CONDUCTING A STEAM GENERATOR INSPEC... NOTARIZED 10/02/78.

PLANT NAME: TURKEY PT #3

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LICENSE.
(DISTRIBUTION CODE A001)

FOR ACTION: BR CHIEF ~~ORDM~~ DC**W/7 ENCL

INTERNAL: REG FILE**W/ENCL
I & E**W/2 ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
REACTOR SAFETY BR**W/ENCL
EEB**W/ENCL
J MCGOUGH**W/ENCL

NRC PDR**W/ENCL
OELD**LTR ONLY
CORE PERFORMANCE BR**W/ENCL
ENGINEERING BR**W/ENCL
PLANT SYSTEMS BR**W/ENCL
EFFLUENT TREAT SYS**W/ENCL

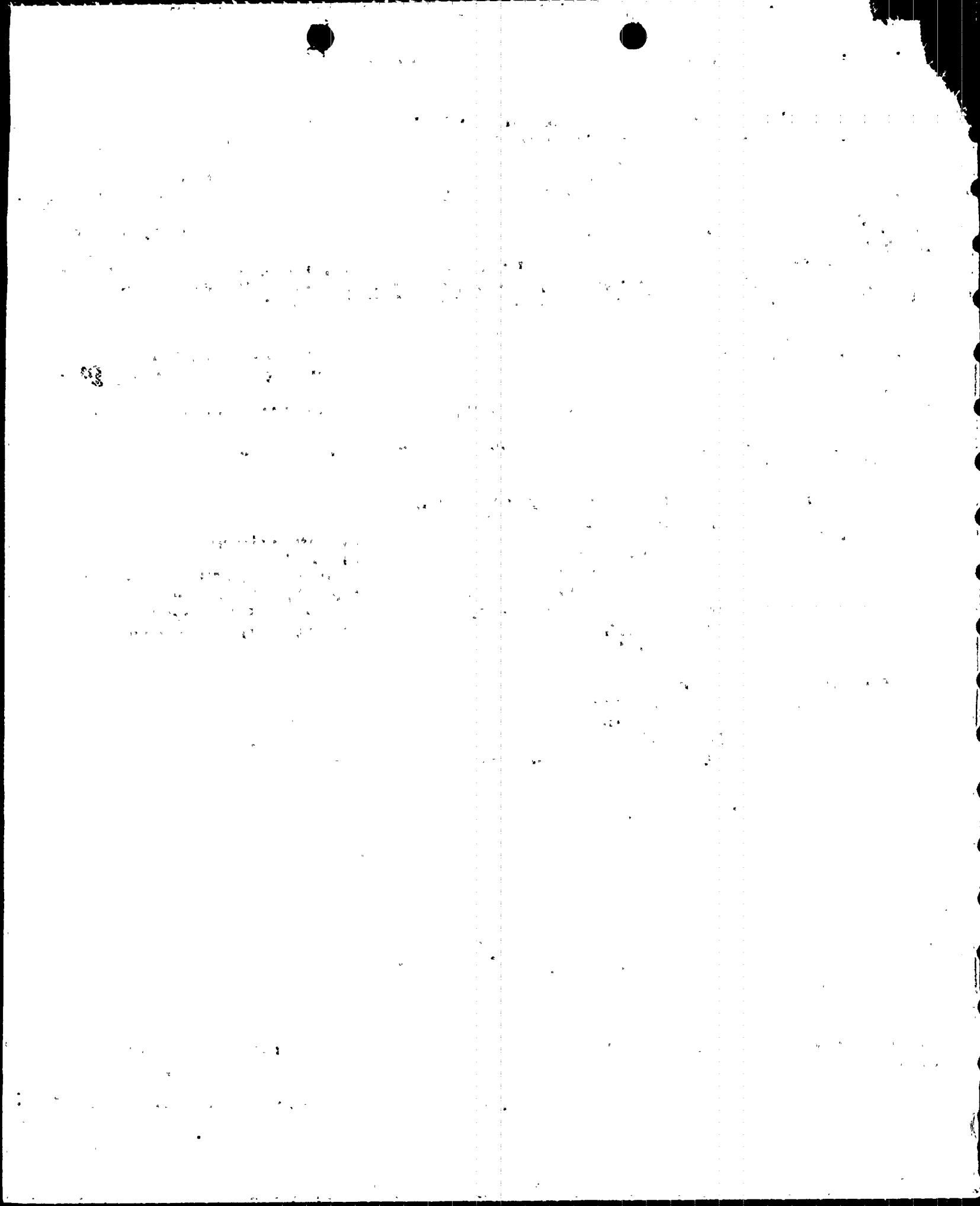
EXTERNAL: LPDR'S
MIAMI, FL**W/ENCL
TERA**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

AA2

DISTRIBUTION: LTR 40 ENCL 39
SIZE: 2P+1P

CONTROL NBR: 780750325

***** THE END *****





REGULATORY DOCKET FILE COPY

October 2, 1978
L-78-323

Office of Nuclear Reactor Regulation
Attn: Mr. Victor Stello, Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Stello:

Re: Turkey Point Unit 3
Docket No. 50-250
Proposed Amendment to
Operating License DPR-31
Supplemental Information

RECEIVED
OCT 6 11 0 57
REGULATORY DOCKET

On September 26, 1978, Florida Power & Light Company requested a license amendment to allow Turkey Point Unit 3 to operate until the next refueling prior to conducting a steam generator inspection. Following discussions with members of your staff we have agreed to perform a steam generator inspection earlier if there is evidence of a significant change in the condition of the steam generator.

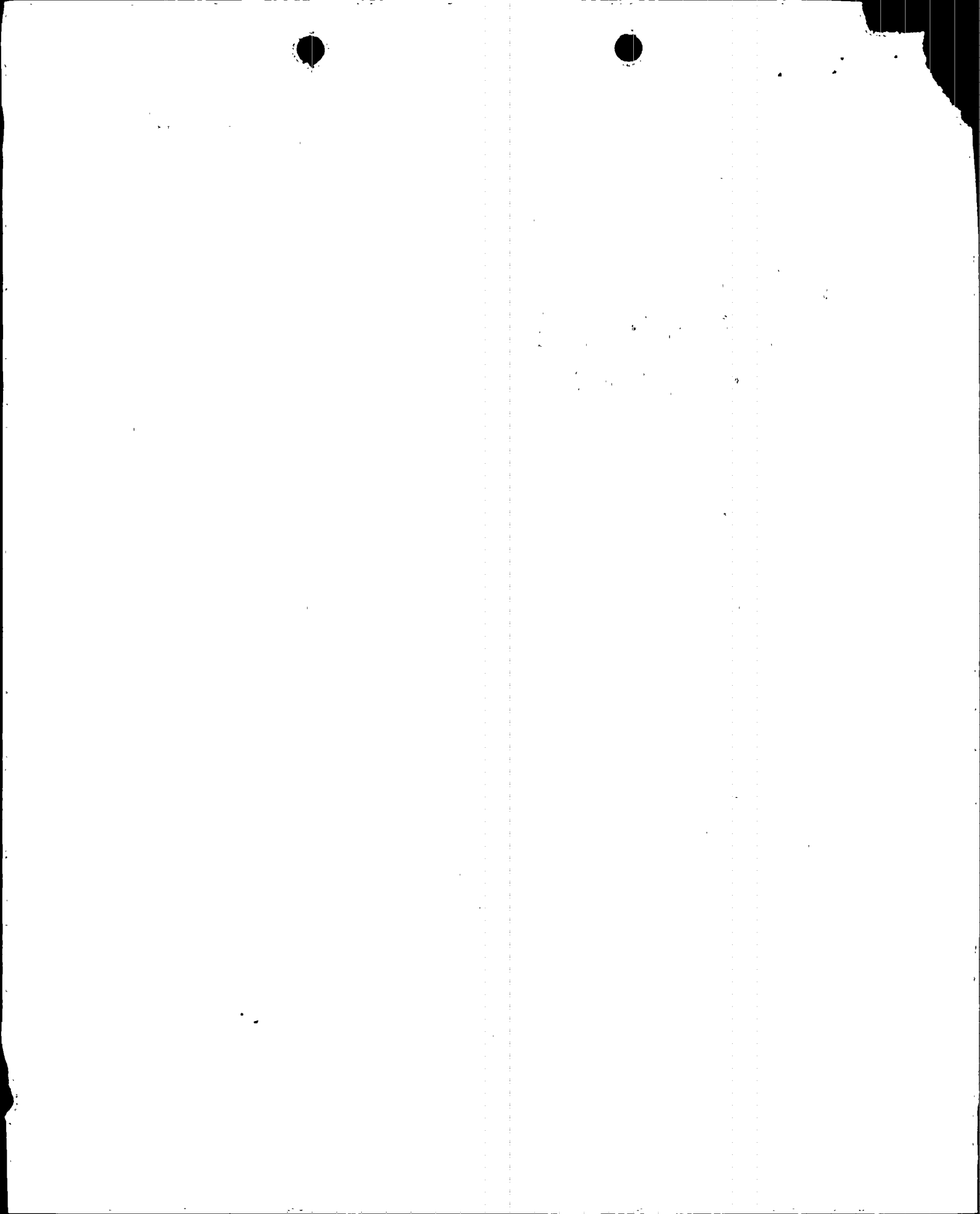
We request that paragraph E5 of operating license DPR-31 be revised to read as follows:

E. Steam Generator Inspections

5. In order to perform an inspection of the steam generators, Unit No. 3 shall be brought to the cold shutdown condition within twelve equivalent full-power months of operation from February 1, 1978, unless (1) an inspection of the steam generators is within this twelve month period as a result of the requirements in 2, 3 and 4 above, or (2) an acceptable analysis of the susceptibility for stress corrosion cracking of tubing is submitted to explicitly justify continued operation of Unit No. 3 beyond the authorized twelve equivalent months of operation. Any analysis justifying continued operation must be submitted at least 45 days prior to the expiration date of the authorized twelve equivalent months of operation.

780750325

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A001/s*
3/40



Mr. Victor Stello, Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
October , 1978
Page 2

The following limits are in effect until the completion of a next steam generator reinspection following the effective date of this amendment.

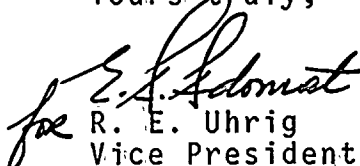
- a. In the event that the .3 GPM limit specified in E.2. above is exceeded, a gaging inspection shall be performed in the affected steam generator. Preventative plugging will be performed in the affected steam generator as indicated by the gaging inspection.
- b. In the event that the two leak in twenty day limit as described in item E.3. above is exceeded, inspection of the Unit 3 steam generators will be performed.
- c. NRC permission will be obtained prior to returning to power operation following an inspection described in "a" or "b" above.

As indicated in our submittal of September 26, 1978, we assumed a maximum of 0.05 GPM additional leakage per restricted tube within the 17½% strain boundary (at 12 EFPM) during a postulated main steam line break. The maximum estimated leakage is based on Westinghouse experiments with a dent at one tube to tube support plate intersection.

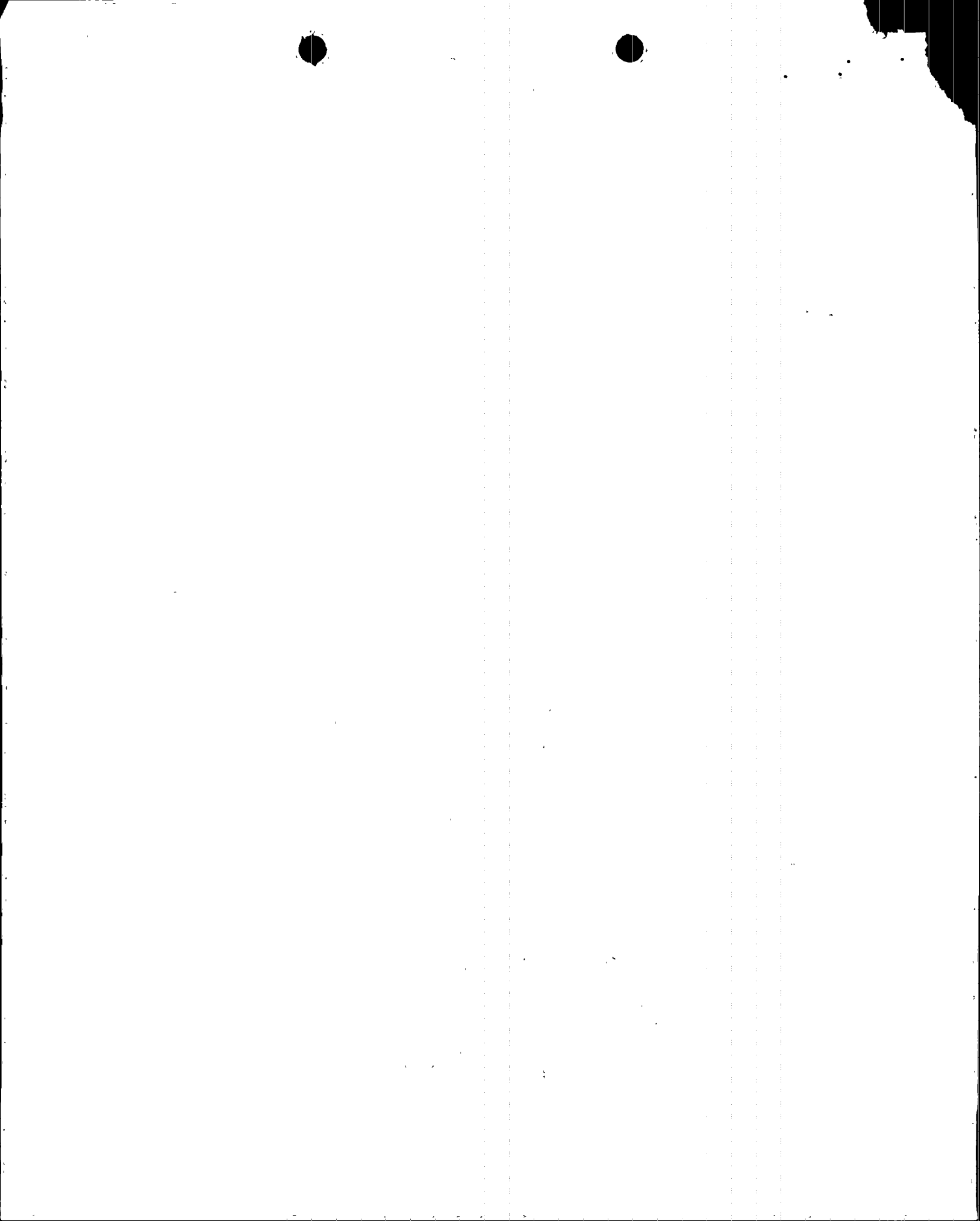
Although at any one time several of the tube to tube support plate intersections may be dented, our inspections show that one intersection leads the others in amount of degradation. (See figure 1-3 in our submittal of June 9, 1977, L-77-173, of Turkey Point 4 steam generator inspection results). We expect that the most severely dented intersection would leak and, therefore, be detected prior to the other intersections degrading to the point where they would develop a leak following a postulated MSLB. For these reasons our use of the 0.05 GPM leakage estimate per tube is valid.

If you should have any question in this matter please feel free to call me.

Yours truly,


R. E. Uhrig
Vice President

cc: Robert Lowenstein, Esquire
J. P. O'Reilly, Region II



RECEIVED
FEBRUARY 1962
U.S. DEPARTMENT OF AGRICULTURE
WASHINGTON, D.C.